# Designing Fusion Machines for High Availability

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## What will DEMO look like?

# High availability is a key ingredient in defining the DEMO configuration and achieving fusions economic goal

- COE is proportional to (Availability)<sup>-1</sup>
- Rapid removal/replacement of limited-life in-vessel components is a necessary condition for high availability.
- DEMO will need to show that a fusion plant can operate with high availability, as its last step before full-scale electricity production.
- Availability drives *fundamental* design choices for DEMO.



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# ITER's port-based maintenance is incompatible with high availability



Final Report of the European Fusion Power Plant Conceptual Study, April, 2005

# High availability designs require large openings to remove and replace large in-vessel modules

#### Multiple approaches have been studied, illustrating tradeoffs.

**U.S. ARIES-AT** 



JAEA, DEMO

# Corridor

#### EU multi-module concept



16-port vertical maintenance

16 radially extended TF coils Toroidal segmentation 16 modules 16-port horizontal removal 12 radially extended TF coils Toroidal segmentation 12 modules 4-port horizontal removal

16 tight-fitting TF coils Toroidal & radial segmentation 64 modules 16-port vertical removal

### Heating systems, diagnostics and services surround a fusion device

#### - challenging the horizontal maintenance option



# As with ITER, vertical installation will be used to assemble DEMO – setting the stage for vertical maintenance



Assembly within the tokamak pit

ITER tokamak building



#### The EU vertical maintenance design represents one option under consideration for DEMO

#### The multi-module maintenance concept incorporates 16 tight fitting TF coils with 64 split blanket segments.

Designed for a Singlenull plasma





Vertical bending bars

Blanket segments attached inclined bending bars and toroidal lower rings.

Toroidal Mid Rings Large ports cut the continuity of a poloidal blanket module





#### A second vertical maintenance option is introduced with enlarged TF coils



## A graded TF design helps to offset the cost of the enlarged coil size



#### In-vessel segmentation concept provides alignment, labyrinth gap shielding and disruption load support



Semi-permanent inboard shield used for alignment, disruption load support and shielding for gaps between modules

## **S. Korea Fusion DEMO**



#### K-DEMO

- 6.8-m R<sub>0</sub>
- 200 600 MW P<sub>net</sub>
- Steady State



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# **K-DEMO incorporates high-availability** design features



#### Magnetic system components evolved from ITER experience



ABORATORY

#### K-DEMO Poloidal Field coil arrangement satisfies performance goals with large vertical openings





R = 6.8 m, DN a = 2.1 m  $\kappa_x = 2.0$  $\delta_x = 0.625$ B<sub>T</sub> = 7.4 T



#### **Operating Space**

Low  $P_{elec} \sim 200-300 \text{ MW}$  $\beta_N^{\text{total}} \leq 3.0$ Ip = 11-12 MA High P =  $\approx 500-600 \text{ MW}$ 

### A test cell built to initially assemble the device also accommodates vertical maintenance



#### Device core shown integrated within a test cell

Space allows replacement of half of the blanket modules simultaneously





# To achieve DEMO availability goals....

A rich set of designs need to be established to maximize the chance of success – no solution is perfect.

The vertical maintenance design being developed for K-DEMO is showing promise...

The EU vertical and the JA 4-port horizontal maintenance concepts also have advantages.

Further developed integrated designs of all options is needed to ensure that high availability can be achieved, compatible with component requirements and cost objectives.



## A Pilot Plant can demonstrate net electricity and Prototype a DEMO maintenance scenario

#### Providing an intermediate step with reduce risks on the path to DEMO

#### K-DEMO 6.8-m device

$$\begin{split} & \mathsf{P}_{elec} \sim 200\text{-}600 \text{ MW}, \text{ TBR } > 1 \\ & \mathsf{P}_{fus} = \textbf{2181} \text{ MW}, < \mathsf{W}_{n} > 2.09 \text{ MW}/\text{m}^{2} \end{split}$$



#### PPPL 4.0-m AT Pilot Plant

Q<sub>engr</sub> ≥ 1, TBR > 1 P<sub>fus</sub> = **674** MW, < W<sub>n</sub> > 2.2 MW/m<sup>2</sup>





# Summary

- Rapid removal and replacement of limited-life in-vessel components is a necessary condition for high availability.
- Success depends on tradeoffs among some very fundamental machine design choices:
  - Number and size of TF coils.
  - > In-vessel segmentation scheme and defined access for module removal.
  - Provision for ease of alignment and gap shielding that is compatible with plasma disruption support.
  - Auxiliary interfaces that enhances in-vessel module removal.
  - Routing of in-vessel services that fosters ease of maintenance.
  - Defined poloidal field coil arrangement compatible with large openings for maintenance access.
- A range of concepts need to be studied and compared.
- Prototyping a DEMO-relevant maintenance concept in an intermediate (pilot-scale) fusion device also should be considered.