### **OVERVIEW OF IGNITOR**

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major radius	R <sub>0</sub>	1.32 m
minor radius	$a \times b$	0.47×0.86m
aspect ratio	Α	2.8
elongation	K	1.83
triangularity	δ	0.4
toroidal field	B <sub>T</sub>	≲ 13T
toroidal current	I <sub>p</sub>	≲ 11 −12 MA
maximum poloidal field	$B_{p,max}$	≲ 6.5 T
mean poloidal field	$\frac{=}{B_p} \equiv I_p / 5\sqrt{ab}$	≲ 3.44 –3.75 T
poloidal current	$I_{\theta}$	≲9MA
edge safety factor ( $I_p \simeq 11 \text{ MA}$ )	$q_{\psi}$	3.6
confinement strength	$S_c \equiv I_p \overline{\overline{B}}_p$	38 – 45 MA·T
plasma volume	V <sub>0</sub>	$\simeq 10 \text{ m}^3$
plasma surface	So	$\simeq 34 \text{ m}^2$
ICRF heating ( $\simeq$ 140 MHz)	P <sub>RF</sub>	18 – 24 MW

## Table 1: Ignitor Reference Design Parameters

Plasma Current $I_p$	11 MA
Toroidal Field $B_T$	13 T
Central Electron Temperature $T_{e0}$	11.5 keV
Central Ion Temperature $T_{i0}$	10.5 keV
Central Electron Density $n_{e0}$	$9.5 \times 10^{20} \text{ m}^{-3}$
Central Plasma Pressure $p_0$	3.3 MPa
Alpha Density Parameter $n_{\alpha}^*$	$1.2 \times 10^{18} \text{ m}^{-3}$
Average Alpha Density $\langle n_{\alpha} \rangle$	$1.1 \times 10^{17} \text{ m}^{-3}$
Plasma Stored Energy W	11.9 MJ
Ohmic Power $P_{OH}$	11.2 MW
ICRF Power $P_{ICRH}$	0
Alpha Power $P_{\alpha H}$	19.2 MW
Bremsstrahlung Power <i>P</i> <sub>brems</sub>	3.9 MW
Poloidal Beta $\beta_p$	0.2
Toroidal Beta $\beta_T$	1.2 %
Central $q(\psi) q_0$	~ 1.1
Edge $q_{\psi}$	3.5
Bootstrap Current <i>I</i> <sub>bs</sub>	0.86 MA
Energy Confinement Time $\tau_{E}$	0.62 sec
Alpha Slowing Down Time $ au_{lpha,sd}$	0.05 sec
Average Effective Charge $\langle Z_{eff} \rangle$	1.2

 Table 2 Example of Plasma Parameters at Ignition

 $\langle \rangle$  = volume average

$$\beta_{p} = 2\mu_{0} \langle p \rangle / \overline{B}_{p}^{2}(a)$$
  

$$\tau_{E} = W / (P_{OH} + P_{\alpha} + P_{ICRH} - dW/dt)$$
  

$$n_{\alpha}^{*} = n_{D} n_{T} \langle \sigma v \rangle \tau_{\alpha,sd}$$
  

$$\tau_{\alpha,sd} = 0.012 T_{e0}^{3/2} (\text{keV}) / n_{e0} (10^{20})$$



Examples of operating scenarios.



Equilibrium configuration for 11 MA and 13 T, as evaluated by the EQUISL code, with



Example of an equilibrium configuration for 10 MA and 13 T, as evaluated by the EQUISL code, with a double X-point laying just outside the first wall







Time evolution of a discharge with 11 MA and 13 T simulated by the JETTO code, showing a) temperature, b) density, c) powers in MW and the parameter  $Q_{\alpha}$ . Ignition is marked by the vertical dotted line.



Confinement time at ignition as a function of the peak temperature, for a range of density and temperature profile factors. The plasma stored thermal energy is  $W = 3\int dV n_e T$ , the fusion power is  $P_{\alpha} = \varepsilon_{\alpha} \int dV n_D n_T \langle \sigma v \rangle_{fus}$ ,  $T_e = T_i$ , and  $n_D = n_T = n_e/2$ .



Normalized plasma pressure profiles at ignition. The dashed and dotted curves represent the analytical fit of the four profiles.



Thermal load distribution under ignition condition for different radial shifts.



# TRINO COMBINED-CYCLE POWER STATION









**IGNITOR MAGNET SYSTEM AND MACHINE STRUCTURE** 

- Introduction
- Ignitor is a high field compact machine proposed and designed to achieve ignition in D-T plasmas. The machine has been conceived as completely integrated system of its major components (toroidal field system, poloidal field system and plasma chamber).
- The toroidal field magnet is made of 12 modules, each including two toroidal field coils (TFC), contained by 4 C-Clamp elements.
- The structural performance of the machine relies on an optimised combination of "wedging", in the TFC inboard legs and in the outboard of the C-Clamp, and "bucking" between the TFC and the central solenoid (CS).
- The inboard legs of the TFC are preloaded to resist the vertical components of the Lorenz forces. The preload is applied through the upper and lower parts of the C-Clamp (the C-clamp "noses") by means of bracing rings (passive system) and an electromagnetic radial press (active system).



IGNITOR machine cross section

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• The primary function of the CS is to produce most of the magnetic flux variation needed to drive the plasma current according to prescribed scenarios. The radial subdivision of the CS into two concentric groups has been adopted to maximise the magnetic flux variation while keeping the coils within their thermal and mechanical limits. The axial subdivision in four layers is required to match the plasma shape with the first wall profile and to make the coil manufacturing easier; it also provides the CS with the suitable flexibility to avoid localized stresses resulting from the fact that the stiffness of the inner leg of the Toroidal Field Coils (TFCs) changes continuously and significantly along the height.	most of the magnetic flux variation needed to drive the os. The radial subdivision of the CS into two concentric netic flux variation while keeping the coils within their livision in four layers is required to match the plasma e coil manufacturing easier; it also provides the CS with es resulting from the fact that the stiffness of the inner	continuously and significantly along the height. $\prod_{m}$	

IGNITOR MAGNET SYSTEM AND MACHINE STRUCTURE

Central Solenoid System (CS)

330

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Winding of the conductor for the prototype of the innermost coil of the central solenoid.



Completed prototype of the innermost coil of the central solenoid.





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The toroidal field coil prototype.



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Finished C-clamp.

IGNITOR MAGNET SYSTEM AND MACHINE STRUCTURE

Radial press system

- The TFCs load bearing capability is enhanced by two radial press systems (upper and lower). Each one consists of two loading system acting both on the C-Clamp "nose": the mechanical permanent preloading and the active electromagnetic press.
- The first system envisages a radial bracing ring and 48 wedges, located between the ring and the C-clamp nose ; the ring reacts against the wedges applying a radial inward force of 180 MN at 30 K. The second loading system consists of an electromagnetic press producing a nominal force of 193 MN in the radial direction (redundant force of 25% is available). Supporting beams keep the press systems in position and withstand vertical electromagnetic loads. The design is such to allow the C-Clamp nose movement and thermal expansion of the coils.
- The radial electromagnetic press is made of two concentric coils (inner coil 210x210 mm<sup>2</sup>; outer coil 210x275 mm<sup>2</sup>) made of oxygen free copper (OFHC). The inner coil pushes the C-clamps "nose" inward through a steel tapered spacer (0.72 mm); the outer coil is fitted with a reinforcing ring. The insulation system is made of glass fabric plus kapton tapes; the coil is vacuum impregnated with an epoxy resin.



Radial press system cross section

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Plasma chamber supports design

- Radial supports (10 off) are acting in both versus (centripetal and centrifugal) with high stiffness. The end of radial ports are connected to the C-Clamp structure. This connection will be free to allow for thermal movement; only during the pulse the connection will be locked by clamping sleeves hydraulically operated. Radial supports are designed to resist up to a maximum of 3.5 MN radial force. The stiffness of the plasma chamber wall will contribute to share the radial loads with the other port. A structural analysis is needed to assess the loads values.
- Lateral supports between radial ports and C-Clamp are needed to resist out of plane forces due to disruptions as well as to accommodate thermal expansion. Each of this support can react to a force normal at the port axis about 0.75 MN.



Radial support.



Finished sector of the vacuum vessel.



First Wall

• To withstand the heat and the electromagnetic loads in normal operating conditions and disruptions.

• To cover with tiles the entire inner surface of the plasma chamber, so that the first wall (FW) basically works as well as extended limiter.

• To reach a position accuracy of:

-  $\pm$  0.1 mm between adjacent tiles, on the same tile carrier;

-  $\pm$  1 mm overall assembly tolerances with respect to their theoretical position.

• No active cooling is required.

• During normal operating conditions:

- operating temperature at plasma start-up T=293 K

- maximum thermal load Q=1.25 MW/m<sup>2</sup>, for t=

4.0 s

First wall poloidal cross section

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