## Transport study in reactor-relevant regime on JT-60U towards advanced steady-state tokamak operation



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# Enhanced collaboration with domestic and foreign universities/laboratories

• Many collaborators from domestic and foreign universities and laboratories contribute to the JT-60 program.

#### In JAPAN



#### From abroad

ASIPP(China), Ecole Polytech. (Switzerland), EFDA-JET (EU), GA (USA), loffe Inst. (RF), KBSI(Korea), KFA Juelich (Germany), Kurchatov Inst. (RF), LANL (USA), MPI-Garching (Germany), MIT (USA), ORNL (USA), PPPL (USA), SWIP (China), TRINITI(RF), U. Stractclyde (UK)



#### Introduction

JT-60U

- ITER Physics R&D
- Advanced Tokamak Concepts for ITER & DEMO
  - High integrated performance;
    - high values of  $\beta_{\text{N}},$  HH  $_{\text{y2}},$  f  $_{\text{BS}},$  f  $_{\text{CD}},$  n/n  $_{\text{GW}},$  fuel purity, P  $_{\text{rad}}/\text{P}_{\text{abs}}$

 $\begin{array}{l} \text{High } \beta_p \text{ mode plasma} \\ \text{(weak positive shear)} \\ \text{Reversed shear plasma} \end{array} \right) \quad \text{with internal transport} \\ \text{barrier (ITB)} \end{array}$ 

- Dominant electron heating (T<sub>e</sub>>T<sub>i</sub>) and small central fueling;
  - T<sub>e</sub> ITB formation
  - Heat and particle (including impurity) transport Sustainment/degradation of ITB and confinement ? Heavy impurity accumulation ?

Development of ECRF (110 GHz) and N-NB systems ECRF : 3 MW for 2.7 s (4 units of gyrotron) N-NB : 6.2 MW for 1.7 s (381 keV)

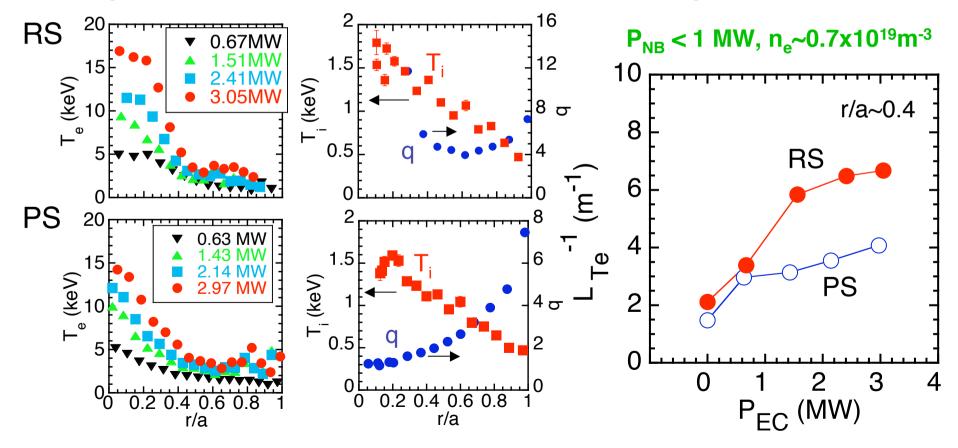


#### OUTLINE

- T<sub>e</sub> ITB formation condition under dominant electron heating by ECRF
- Heat and particle transport in the ITB region under dominant electron heating / small central fueling by ECRF
  - Reversed shear plasma
  - Weak positive shear plasma
- Summary and Future plan

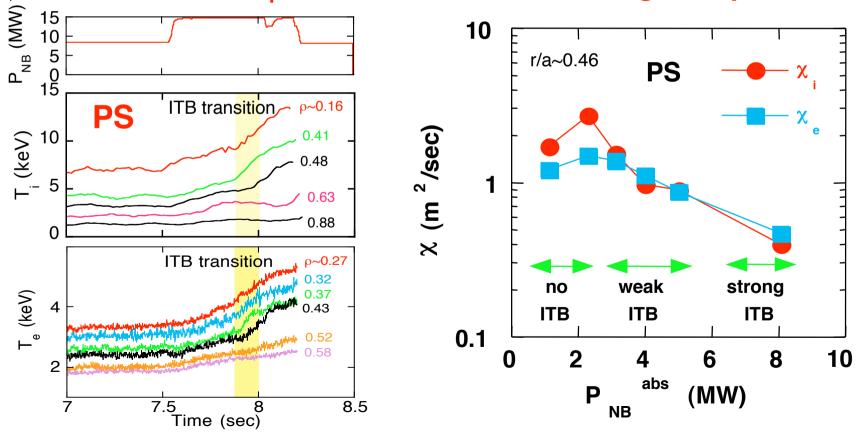
### Strong T<sub>e</sub> ITB is formed without T<sub>i</sub> ITB in RS plasma.

- T<sub>e</sub> ITB is important to improve confinement in ITER/DEMO, where T<sub>e</sub> is expected to be higher than T<sub>i</sub>.
- With no/low P-NB power, strong T<sub>e</sub> ITB is formed in RS plasma (P<sub>EC</sub>~ 1 MW) but not formed in PS plasma (P<sub>EC</sub> up to 3 MW).



#### Strong T<sub>e</sub> ITB is formed with T<sub>i</sub> ITB in PS plasma.

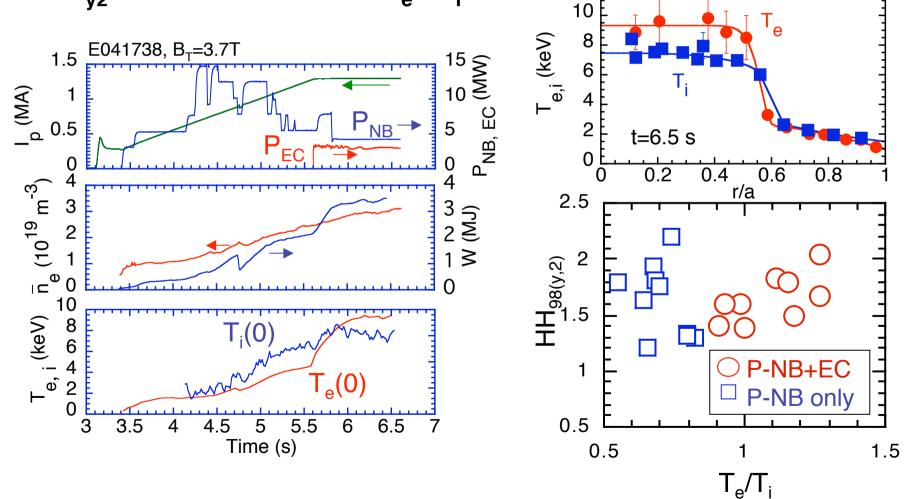
- Formation of strong T<sub>e</sub> ITB requires strong T<sub>i</sub> ITB in PS plasma with high power P-NB heating.
- Electron heat transport strongly correlates with ion heat transport.
  - Sustainment of T<sub>i</sub> ITB under electron heating is important.



## High confinement is obtained with T<sub>e</sub>>T<sub>i</sub> in RS plasma.

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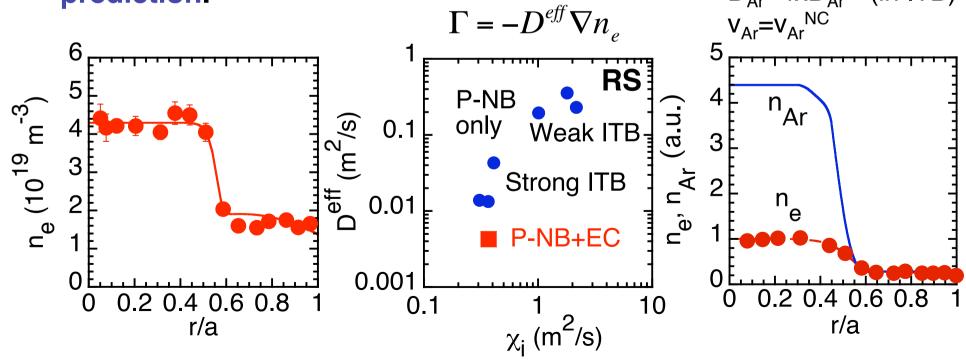
- In RS plasma, strong T<sub>i</sub> and T<sub>e</sub> ITBs are maintained under dominant electron heating (P<sub>ele.</sub>~1.6xP<sub>ion</sub>).
- $HH_{y2}$ ~2 is achieved with  $T_e > T_i$ .



# Strong n<sub>e</sub> ITB is kept and Ar is accumulated even with small central fueling in RS plasma

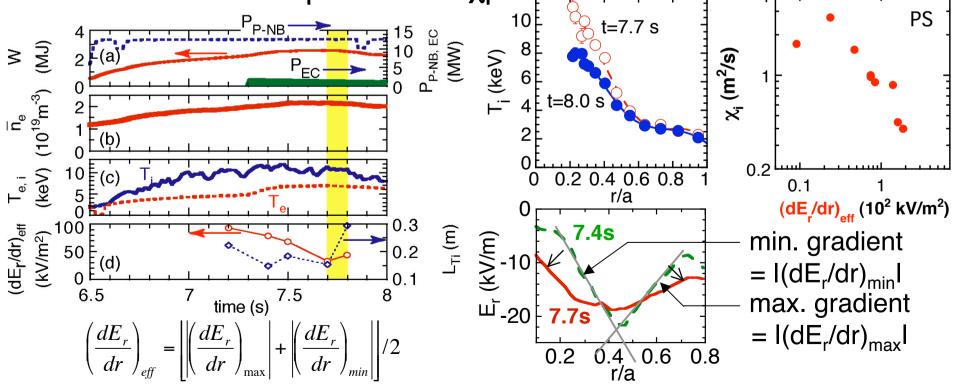
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- A flat density profile is favorable for suppression of impurity accumulation due to small neoclassical inward pinch velocity.
- In RS plasma, strong n<sub>e</sub> ITB is maintained even with small central fuelling and D<sup>eff</sup> is smaller with small central fuelling.
- Ar is accumulated inside the ITB even with small central fueling, although Ar accumulation is weaker than neoclassical prediction.



#### T<sub>i</sub> ITB degrades by injecting ECRF in PS plasma

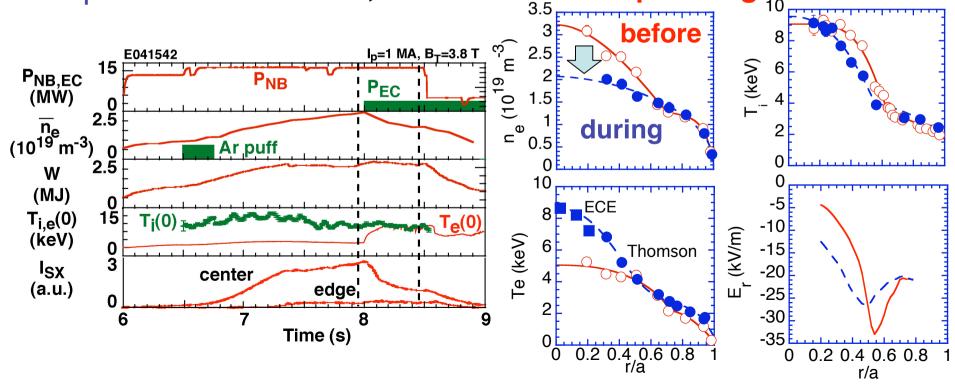
- Toroidal rotation is flattened and decrease of E<sub>r</sub> shear followed by degradation of T<sub>i</sub> ITB is observed.
- Remarkable correlation between effective  $\text{E}_{\text{r}}$  shear and  $\chi_{\text{i}}$  is observed.
- The T<sub>i</sub> ITB degradation is consistent with the correlation between effective E<sub>r</sub> shear and  $\chi_i$ . 12



#### Density profile is flattened by injecting ECRF in PS plasma with Ar puffing

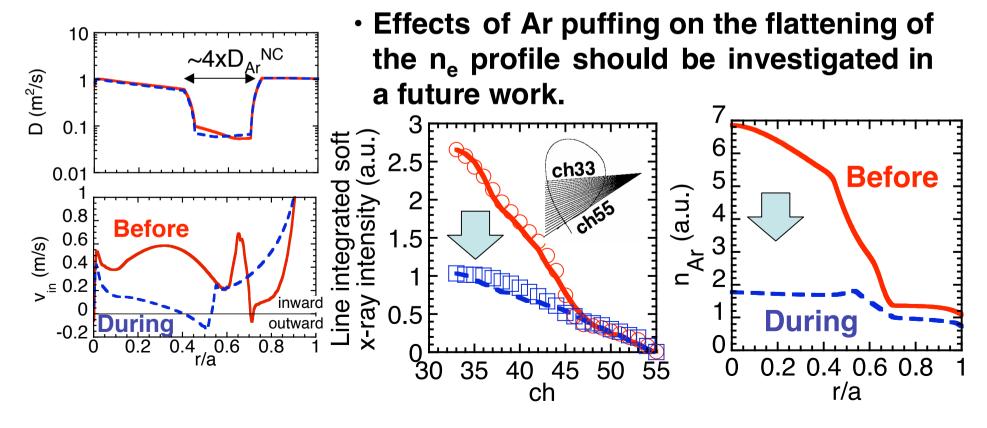
- Large decrease of n<sub>e</sub> is observed with small amount of Ar puffing in PS plasma.
- Density and central soft x-ray signal are drastically reduced by injecting ECRF.
- $\cdot$  n<sub>e</sub> ITB is almost lost, but T<sub>i</sub> ITB is sustained.

• E<sub>r</sub> shear also reduced, however is still kept at high value.



#### Ar is exhausted from the inside of the ITB.

- n<sub>Ar</sub> profile estimated from soft X-ray profile is more peaked by a factor of 1.6 than n<sub>e</sub> profile before ECRF injection and similar as n<sub>e</sub> profile during ECRF injection in PS plasma.
- The reduction of  $n_{Ar}$  is consistent with the reduction of  $v_{Ar}^{NC}$  due to the reduction of  $n_e$  gradient
- $D_{Ar}$  is higher by a factor of 4 than  $D_{Ar}^{NC}$  in the ITB region.



### Summary

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 ITB transport is investigated under dominant electron heating and small central fueling by using ECRF system.

	Reversed shear	Weak positive shear
T <sub>e</sub> ITB formation	•Strong T <sub>e</sub> ITB without T <sub>i</sub> ITB (P <sub>EC</sub> ~1MW)	•No strong T <sub>e</sub> ITB without T <sub>i</sub> ITB (P <sub>EC</sub> <3MW)
Sustainment/ degradation of ITB and confinement	<ul> <li>Sustainment of strong ITBs (T<sub>i,e</sub>, n<sub>e</sub>)</li> <li>High confinement of HH<sub>y2</sub>~2 with T<sub>e</sub>/T<sub>i</sub>&gt;1</li> </ul>	•T <sub>i</sub> ITB degradation •Flattening of n <sub>e</sub> profile and sustainment of T <sub>i</sub> ITB with Ar puff
Heavy impurity accumulation	•Ar accumulation	•Ar exhaust from the inside of the ITB with flattening of n <sub>e</sub> profile

 Further study is necessary with N-NB : Are the effects observed in PS plasma intrinsic to ECRF injection or electron heating ?

#### Main objectives in JT-60U 2003-4 experiment campaign

- Long sustainment (~30-60 s) of high performance plasmas for >τ<sub>R</sub> and ~τ<sub>wall</sub>
   Extension of operation pulse length (15 s -> 65 s)
   NB power : 14 MWx30s
   RF power : ~2 MWx60s
- Sustainment of high  $\beta_N$  (3-3.5) by stabilizing NTM
- Extension to high density towards highly integrated performance