Overview of the Fusion Program at General Atomics

Fusion Power Associates Meeting

Washington, DC

December 2, 2009













GA's target fabrication division designs, develops and produces targets as a partner in the ICF mission



Targets have played a central role in success of NNSA facilities



GA is a leading developer of rep-rated and IFE target supply processes and target related systems



Making foam or fast ignition shells



Robotic assembly



Rapid insertion at Gemini



Inject targets (gas-gun or electromagnetic)

Electrically steer targets



Hit-on-fly demo for target tracking



Fill targets & layer in a cryogenic fluidized bed



GA Theory Program is Organized in Focus Areas to Support OFES Mission

• Turbulence and Transport

- Focus on understanding mechanisms for particle and heat transport in magnetically confined fusion plasmas
- Develop extensive simulation capabilities for nonlinear interactions between magnetized charged particles and electromagnetic waves

Integrated Modeling

- Focus on self-consistent integration of various physical components
- Enable experimental interpretation, model validation and next step design work

Strengths of GA Theory Program:

- Strong collaboration with experiments
- Leaders in numerical simulations using high performance computers
- Extensive international collaborations
- Strong post-doc program



Transport Models (GLF23, TGLF) Derived From GYRO Allow Predictive Transport Simulations Inside H-mode Pedestal

- Newly developed synthetic diagnostic algorithms have been applied to massively
 parallel GYRO simulations of a DIII-D discharge to allow for direct comparisons with
 upgraded beam emission spectroscopy (BES, left) and new correlation electron
 cyclotron emission radiometry (CECE right) diagnostics
 - Simulations used > 3000 processor-hours on jaguar CRAY XT4 machine at NCCS



New Predictive Physics Model (EPED1) for the Pedestal Developed and Tested



- Formation of an edge barrier or "pedestal" allows high performance operation in tokamaks [left]
 - Predicting pedestal height key to optimization of the tokamak
- New EPED1 model combines two constraints to predict height and width of the pedestal
 - Peeling-ballooning stability constraint calculated with the ELITE code [solid line]
 - Simple model of kinetic ballooning mode transport constraint [dashed line]
- EPED1 highly successful in initial tests, including predictions made before a dedicated DIII-D experiment
 - Predicted/Measured pedestal height = 1.02 ± 0.13 (21 DIII-D, 16 JT-60U, 4 JET)



DIII-D is an International Research Program



US Labs

LANL (Los Alamos, NM) LBNL (Berkelev, CA) LLNL (Livermore, CA) ORNL (Oak Ridge, TN) PPPL (Princeton, NJ) SNL (Sandia, NM)

Industries

ALITRON (CA) AWE Comm (Germany) Calabasas Creek (CA) CompX (Del Mar, CA) CPI (Palo Alto, CA) Digital Finetec (Ventura, CA) DRS (Dallas, TX) DTI (Bedford, MA) FAR-TECH, Inc. (San Diego, CA) GA (San Diego, CA) Lodestar (Boulder, CO) NOVA Photonics (Princeton, NJ) SAIC (La Jolla, CA) Spinner (Germany) Tech-X (Boulder, CO) Thermacore (Lancaster, PA) TSI Research (Solana Beach, CA)

Auburn (Auburn, Alabama)

US Universities

Colorado School of Mines (Golden, CO) Columbia (New York, NY) Georgia Tech (Atlanta, GA) Hampton (Hampton, VA) Lehigh (Bethlehem, PA) Maryland (College Park, MD) Mesa College (San Diego, CA) MIT (Cambridge, MA) New York U. (New York, NY) Palomar (San Marcos, CA) Purdue U. (W. Lafayette, IN) SDSU (San Diego, CA) Texas (Austin, TX) UCB (Berkeley, CA) UC Davis (Davis, CA) UCI (Irvine, CA) UCLA (Los Angeles, CA) UCSD (San Diego, CA) U. Arizona (Tucson, AZ) U. New Mexico (Albuquerque, NM) U. Oklahoma (Tulsa, OK) U. Rochester (NY) U. Utah (Salt Lake City, UT) U. Washington (Seattle, WA) U. Wisconsin (Madison, WI)

Europe

CEA (Cadarache, France) CFN-IST (Lisbon, Portugal) Chalmers U. (Göteberg, Sweden) CIEMAT (Madrid, Spain) Consorzia RFX (Padua, Italy) CRPP (Lausanne, Switzerland) EFDA (Belgium) FOM (Utrecht, The Netherlands) Frascati (Frascati, Lazio, Italy) FZ-Jülich (Germany) Helsinki U. (Helsinki, Finland) IFP-CNdR (Italy) IPP (Greifswald, Germany) IST (Lisbon, Portugal) ITER (Cadarache, France) JET-EFDA (Culham, United Kingdom) Kharkov IPT (Ukraine) Max Planck (Garching, Germany) U. Dusseldorf (Germany) UKAEA (Culham, United Kingdom) U. Naples (Italy) U. Rome (Italy) U. Strathclyde (Glasgow, Scotland)

Japan

JAEA (Naka, Ibaraki-ken, Japan) NIFS (Toki, Gifu-ken, Japan) Tsukuba U. (Tsukuba, Japan)

Russia

loffe (St. Petersburg) Keldysh (Udmurtia, Moscow) Kurchatov (Moscow) Moscow State (Moscow) St. Petersburg State Poly (St. Petersburg) TRINITI (Troitsk) Inst. of Applied Physics (Nizhny Novgorod)

Other International

Australia National U. (Canberra, AU) ASIPP (Hefei, China) IPR (Gandhinager, India) NFRI (Daejeon, S. Korea) Nat. Nucl. Ctr (Kurchatov City, Kazakhstan) Pohang U. (S. Korea) Seoul Nat. U. (S. Korea) SWIP (Chengdu, China) U. Toronto (Toronto, Canada)

- 100 institutions worldwide
- 430 scientific authors (2008 IAEA)
 - GA: 120
 - Collab: 310
- Students, post docs, and faculty from 53 universities
 - 18 PhD students
 - 18 Post Docs



DIII-D Mission: Establish the Scientific Basis for the Optimization of The Tokamak Approach to Fusion Energy Production





ELM-Suppression Using the DIII-D Internal Coils Has Led to the Design of Internal Coils for ITER





An International Team Recently Carried Out Experiments on DIII-D Simulating ITER-TBM Error Fields

Michael Schaffer Joseph Snipes Charles Greenfield Valery Chuyanov Alberto Loarte Naouki Oyama Kouji Shinohara Hogun Jhang Kwang-II You Xiang Gao Songlin Liu Yanjing Chen Guoyao Zheng Gabriella Saibene Peter de Vries Tuomas Tala **Filomena Nave** Anti Salmi Oliver Schmitz Marcin Jakubowski Ruth Laengner Henning Stoschus R. Srinivasan R. Narayanan Punit Gohil Don Spong David Gates Jong-Kyu Park Gerrit Kramer

USA Ю USA Ю IO Japan Japan S Korea S Korea China China China China Europe Europe Europe Europe Europe Europe Europe Europe Europe India India USA USA USA USA USA





GA Seeks to Apply its Experience with the ITER CS Model Coil to the ITER Central Solenoid



CS Model Coil Inner Solenoid



CS



GA is the World Leader in ECH Transmission Lines and Components in the 100 GHz Range

JAPAN:









GA has delivered 42 lines / 3 km total

JT-60U: 4 lines

LHD: 2 + lines

TRIAM-1M: 1 Gamma-10: 1

EUROPE:



TCV: 9





FTU: 4



TJ-II: 1





DIII-D: 6



Haystack: 4



KSTAR: 1

ITER: 56 lines 4 km total



The Fusion Development Facility Mission: Show Fusion Can Produce Energy and its own Fuel

- FDF will:
 - Produce significant fusion power (100-300 MW)
 - Demonstrate fusion fuel self-sufficiency
 - Show fusion can produce high grade process heat and electricity
 - Provide a materials irradiation facility to develop low activation, high strength, high temperature, radiation resistant materials.
 - Enable research on high performance, steady-state, burning plasmas for Demo.
 - Öbtain first data on fusion system operation, fuel management, reliability, availability, and maintainability to guide future fusion energy development.

• By operating steady-state with

- Modest energy gain (Q<7)
- Continuous operation for 30% of a year in 2 weeks periods
- High neutron fluence (3-6 MW-yr/m²)



A Fusion Nuclear Science Facility, ITER, Superconducting Tokamaks, and a Materials Test Facility Enable DEMO





Fusion Nuclear Science Needs to Become as Strong as the Other Two Existing Legs of the US Fusion Program





Fusion Nuclear Science Program (FNSP): The Scientific Basis for Fusion Energy Applications



DIII-D and Other Tokamaks Can Solidify the Physics Basis for the FNSF in 2–3 Years

- Required stability values already achieved in 100% non-inductive plasmas in DIII-D (extend pulse length)
- RWM stabilization by rotation
- NTMs already stabilized
- ELMs gone stochastic edge field
- ELMs gone QH mode operation
- Confinement quality required already obtained in long pulse DIII-D plasmas
- Bootstrap fractions already achieved
- Far off-axis LHCD in H-mode
- Pumped, high triangularity plasma
- DIII-D plasma control system
- Power exhaust more challenging than DIII-D and comparable to ITER
- Main challenge is PFC tritium retention



Green = already achieved, Blue = near term, Red = main challenge



It is Time for Fusion

• We are ready to make the transition

from fusion's preparatory era (research with hydrogen and deuterium plasmas)

to fusion's nuclear science era (research with burning DT plasmas)

