Stability and Control of Resistive Wall Modes in Low-Rotation Tokamak Plasmas

EX/7-1Ra: Active Control of Resistive Wall Modes in High Beta Low Rotation Plasmas

A.M. Garofalo*

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EX/7-1Rb: Plasma Rotation and Wall Effects on Resistive Wall Mode in JT-60U

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Presented by
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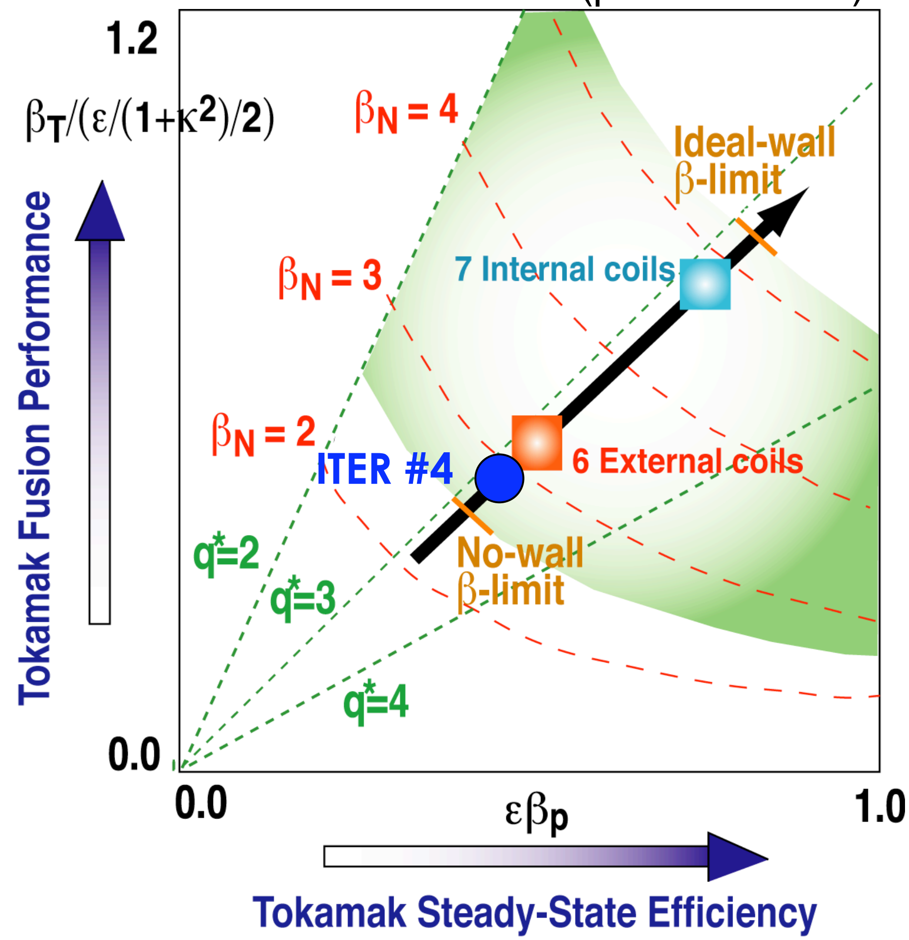
Resistive Wall Mode Stabilization is Needed for Steady State Tokamak Operation at High Fusion Performance

- **ITER Steady-State scenario (#4) requires Resistive Wall Mode stabilization**
  - Target: $\beta_N \sim 3$, above the no-wall stability limit $\beta_N^{no\text{-}wall} \sim 2.5$
- **Sufficient plasma rotation could stabilize RWM up to ideal-wall β_N limit**
- **Present ITER design of external error field correction coils is predicted to allow RWM feedback stabilization if plasma rotation is not sufficient**

VALEN RWM feedback modeling: ITER with blanket (ports covered)



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- **Sufficient plasma rotation could stabilize RWM up to ideal-wall β_N limit**
- **Present ITER design of external error field correction coils is predicted to allow RWM feedback stabilization if plasma rotation is not sufficient**
- **Improved design for RWM stabilization could allow studies of scenarios approaching advanced tokamak reactor concepts, i.e. $\beta_N > 4$**

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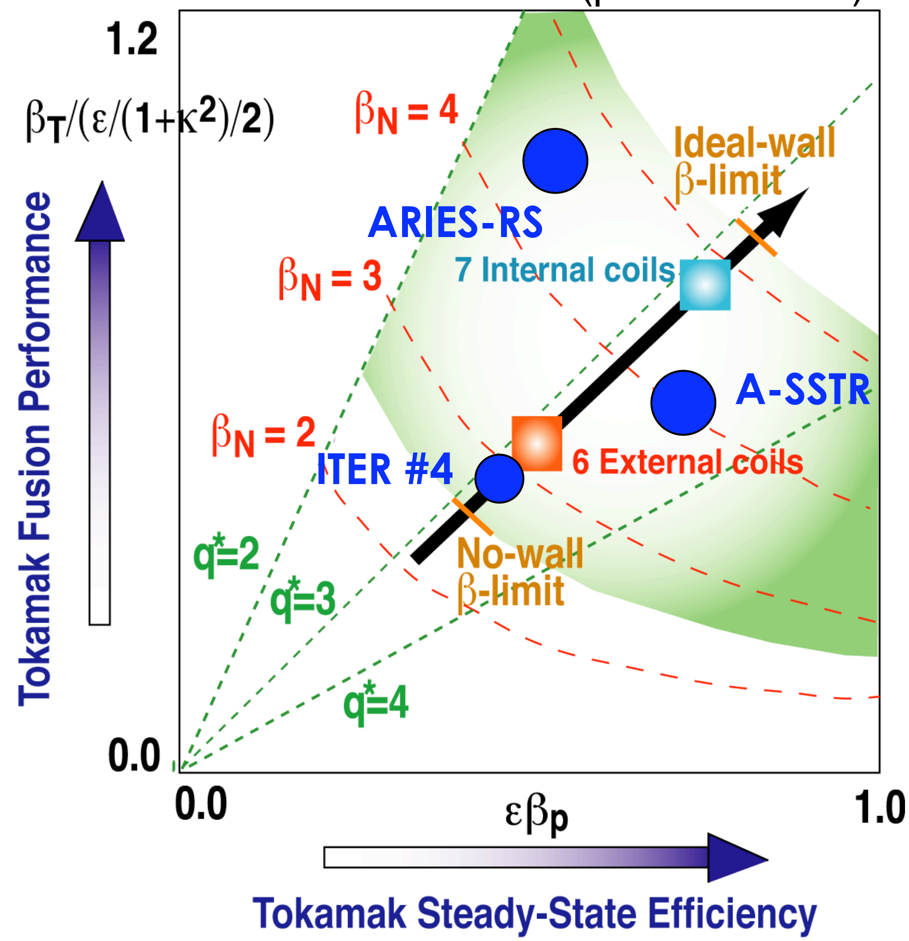
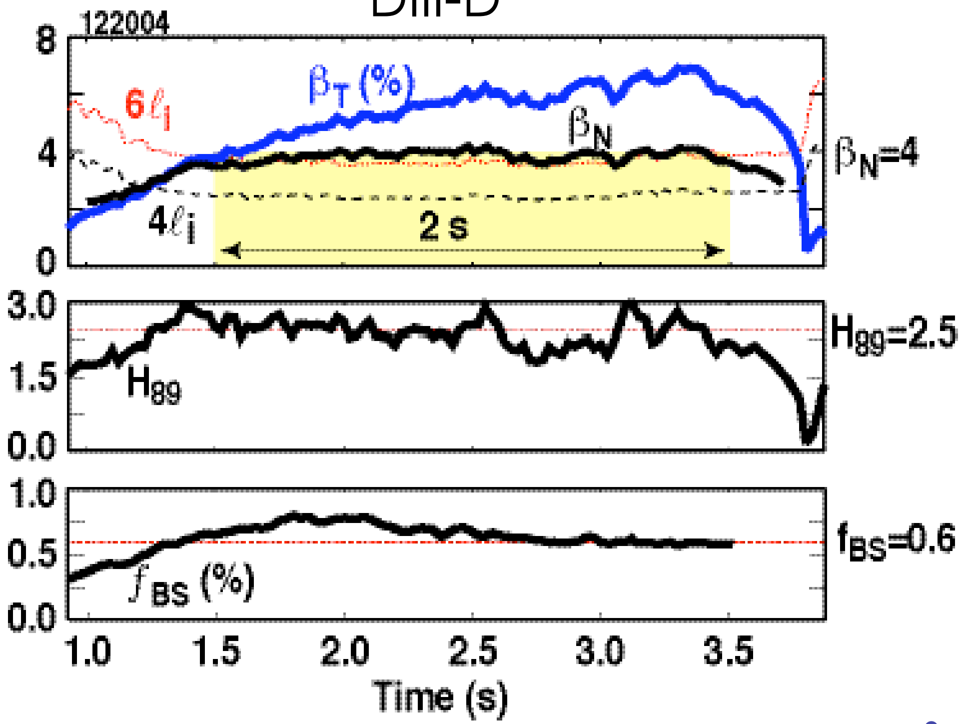
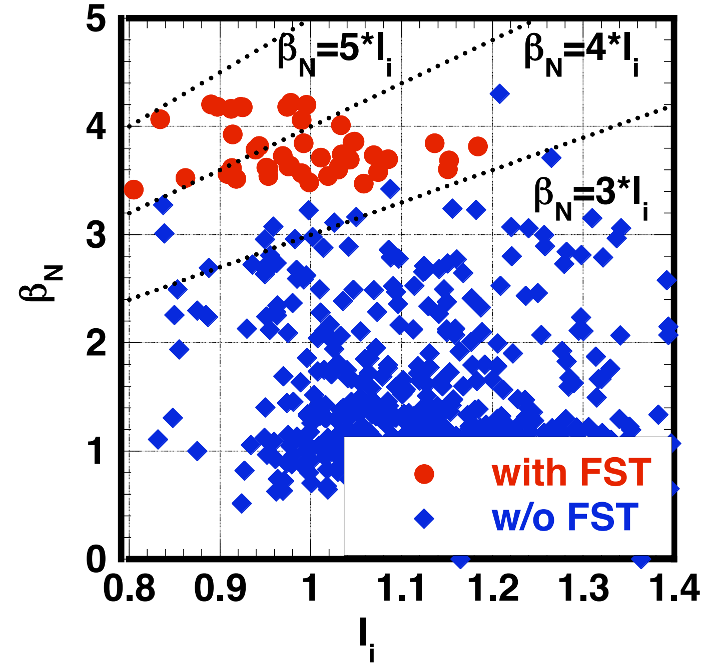


RWM Stabilization by Rotation Allows Demonstration of High Performance Tokamak Regimes

DIII-D



JT-60U



- **High β, β_N, high bootstrap current fraction, high energy confinement sustained simultaneously in DIII-D**
  - RWM feedback -> sustained high plasma rotation

- **High β_N achieved with ferritic steel tiles in JT60U**
  - Reduced ripple loss -> higher confinement and rotation with smaller plasma-wall separation

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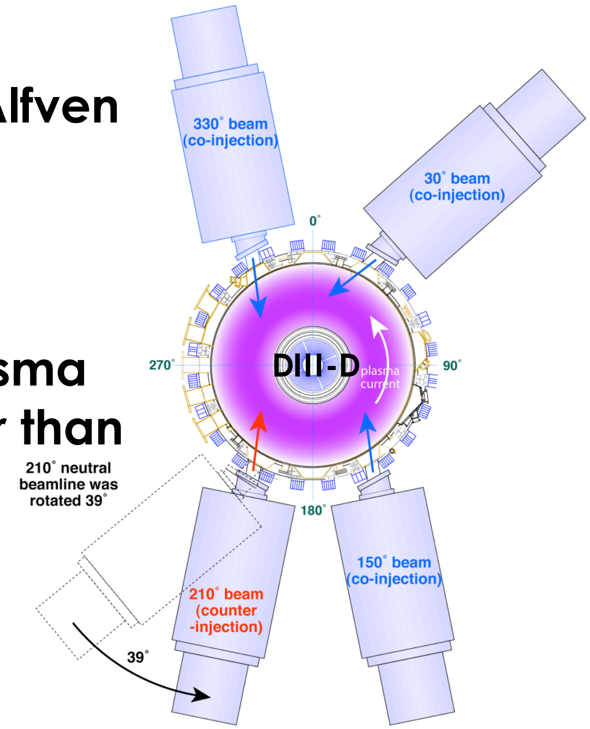
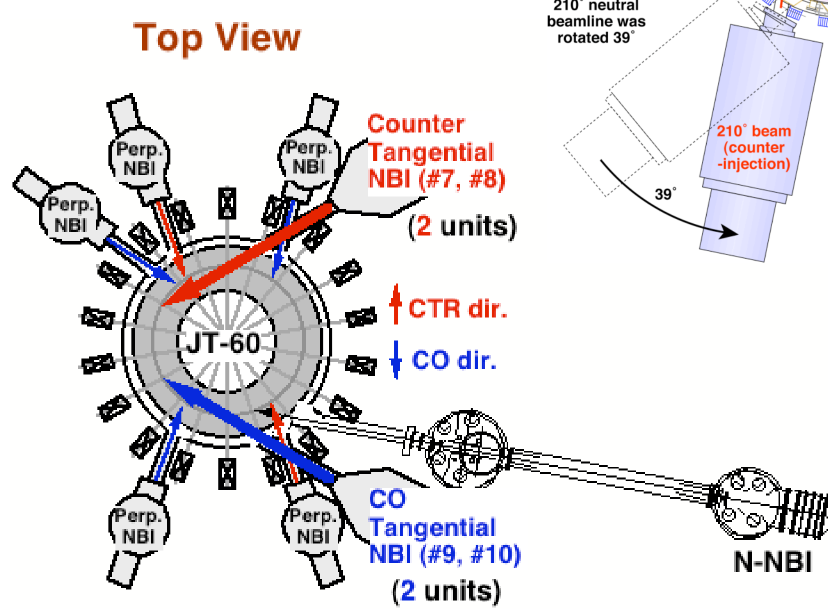
Will RWM Stabilization by Rotation Work in ITER?

- **Until recently, it was believed that RWM stabilization required mid-radius plasma rotation ~O(1%) of the Alfven frequency, Ω_A**
  - This level of rotation may not be realized in ITER
- **Recent experiments using balanced neutral beam injection (NBI) in DIII-D and JT-60U show that the plasma rotation needed for RWM stabilization is much slower than previously thought**
  - ~O(0.1%) of Ω_A
  - Such a low rotation should be easily achieved in ITER
- **Even with sufficient rotation, active feedback may still be needed, but the system requirements could be reduced**





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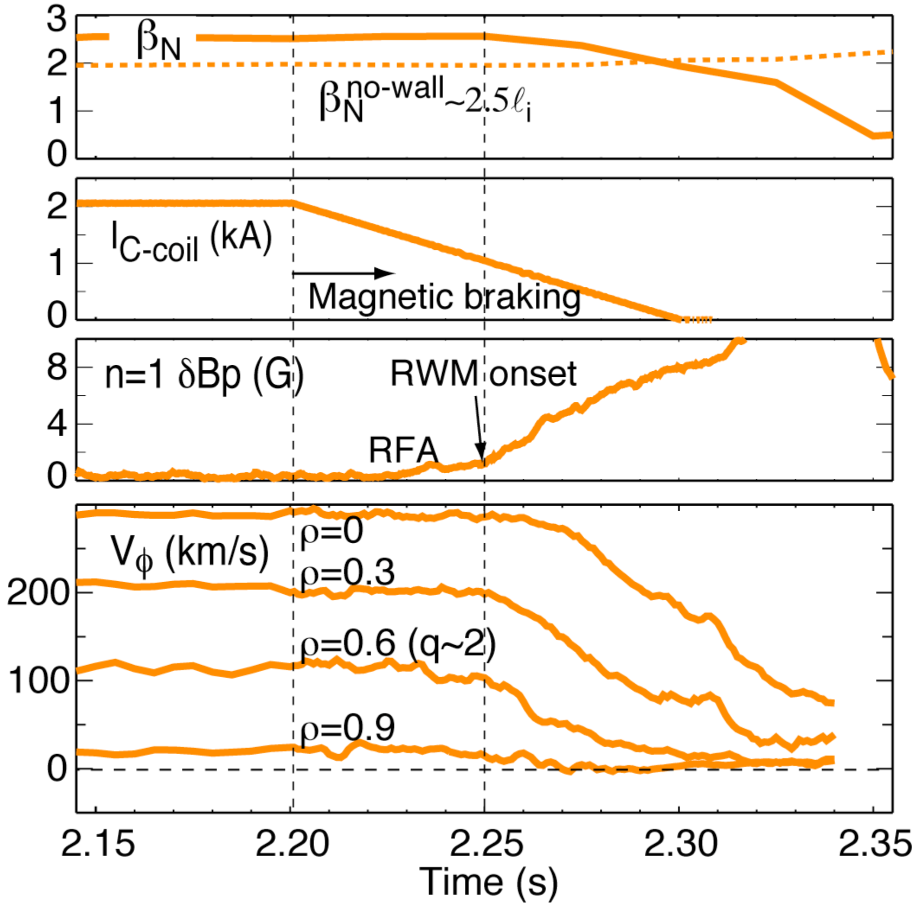
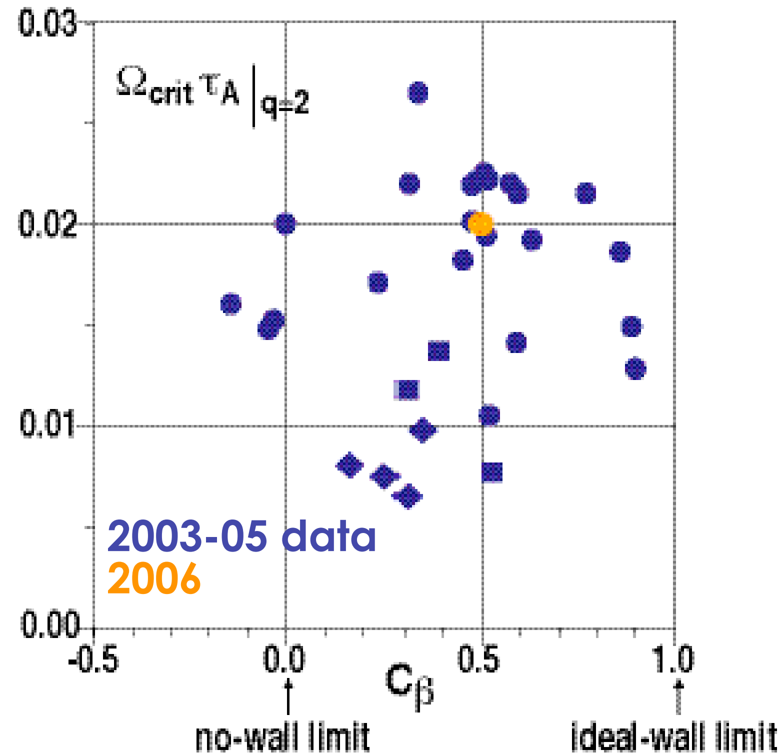
Previously, RWM Rotation Thresholds Were Measured Through Magnetic Braking by n=1 External or Intrinsic Fields

- **DIII-D using only uni-directional NBI:**
  - Magnetic braking is applied by removing the empirical correction of the intrinsic n=1 error field

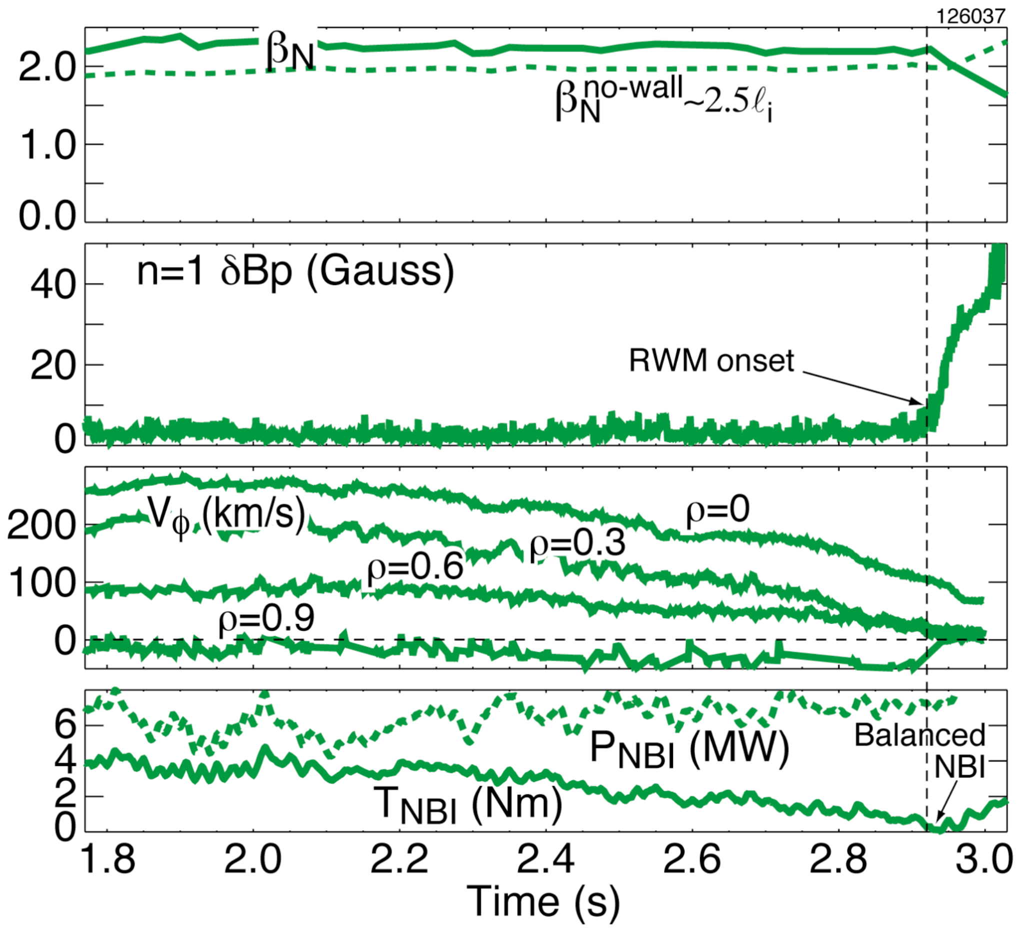


  - Critical rotation frequency Ω_{crit} at q = 2 surface ranges from 0.7 to 2.5% of local Ω_A

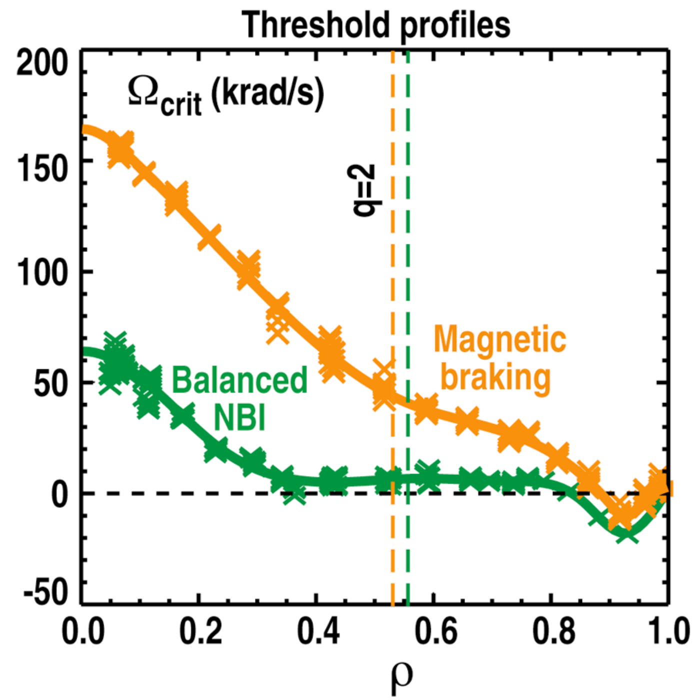


Much Slower Rotation Before RWM Onset is Observed by Reducing the Injected Torque With Minimized Error Fields

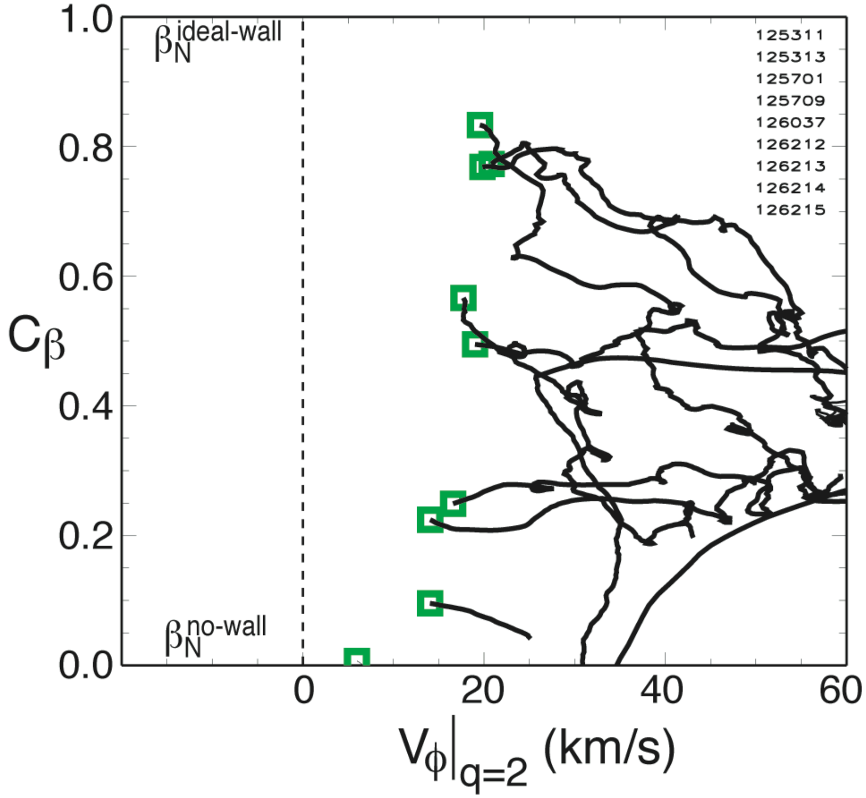
- **DIII-D using a varying mix of co and counter NBI:**

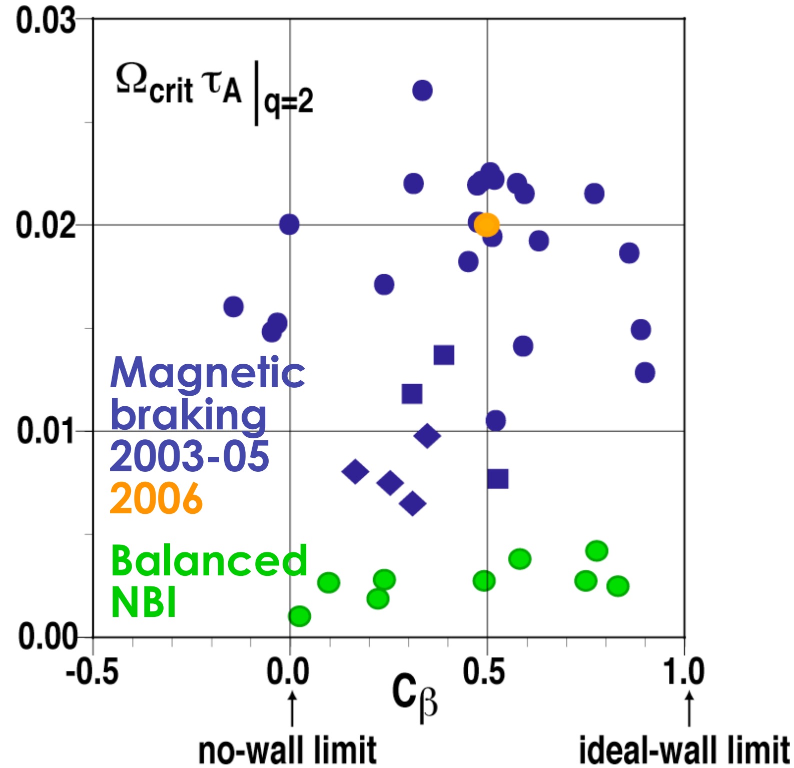


  - Plasma rotation is reduced uniformly for $\rho<0.9$
  - Ω_{crit} at q = 2 is ~10x slower than measured with magnetic braking



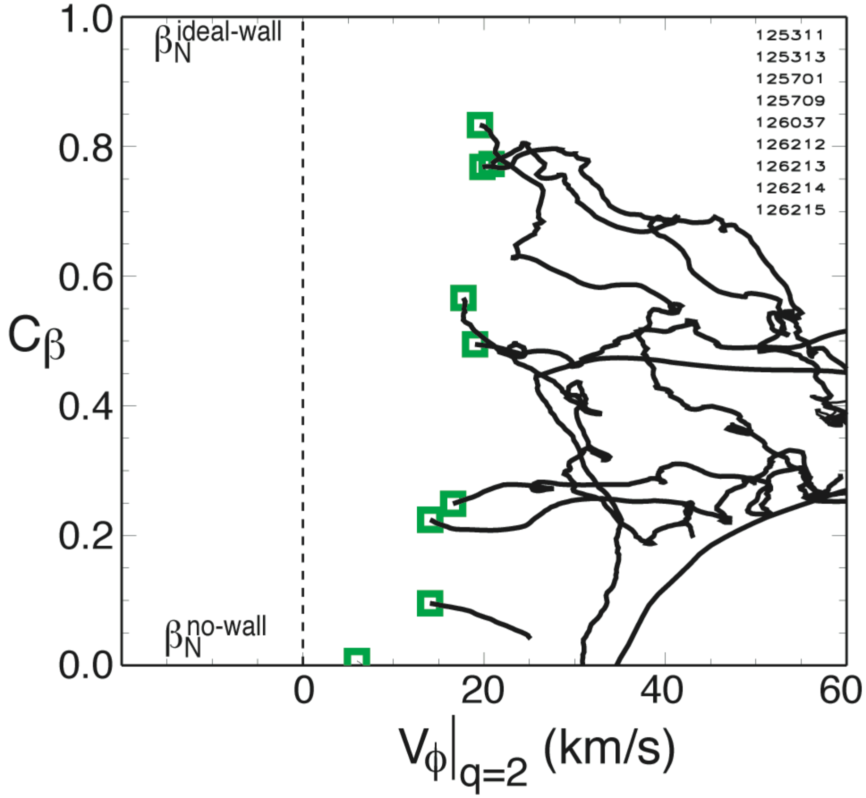
Weak β-Dependence is Observed for Rotation Thresholds Measured With Minimized Error Fields

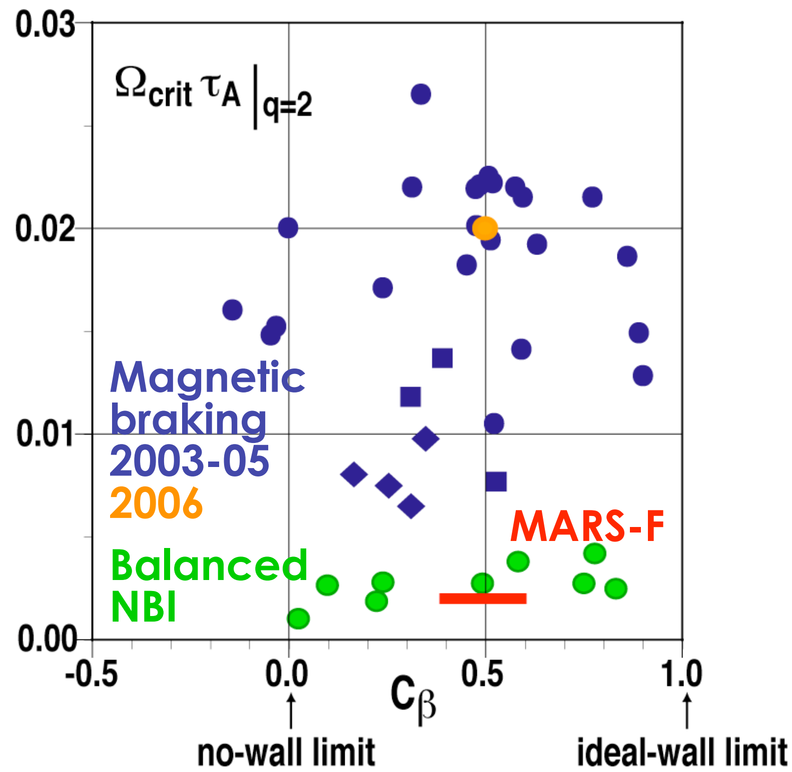




- **RWM onset (❐) observed when V_ϕ at *q*=2 is ~10-20 km/s, or ~0.3% of local V_A**

Weak β-Dependence is Observed for Rotation Thresholds Measured With Minimized Error Fields



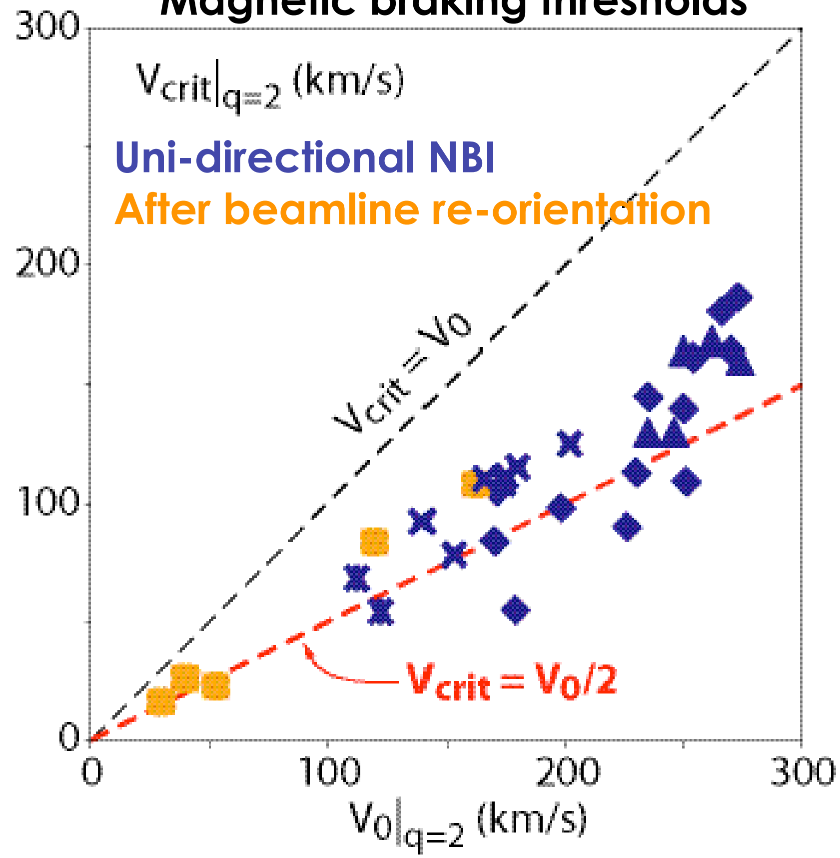


- RWM onset (❐) observed when V_ϕ at q=2 is ~10-20 km/s, or ~0.3% of local V_A
- Ideal MHD with dissipation implemented in MARS-F (kinetic damping model [Bondeson and Chu]) predicts slow rotation threshold for balanced NBI plasmas

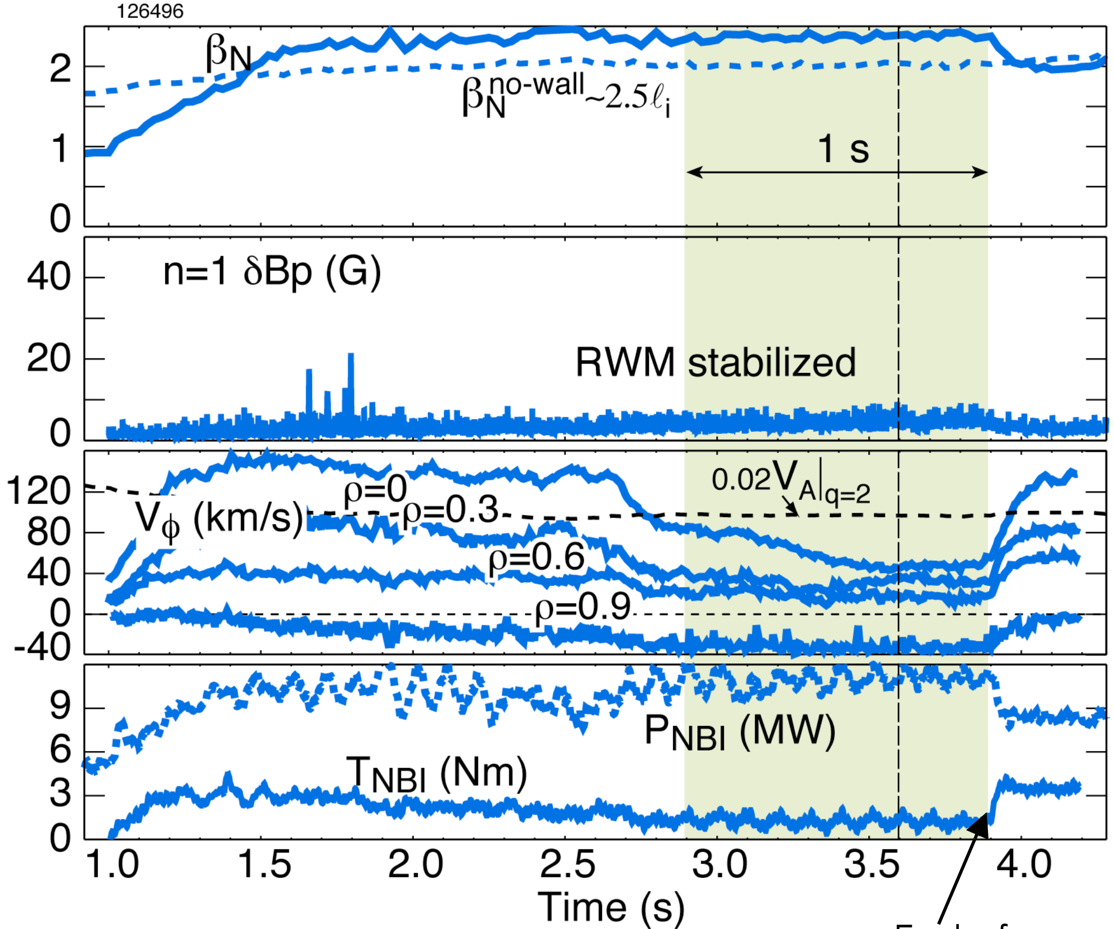
High Threshold Measured With Magnetic Braking May Correspond to Entrance Into Forbidden Band of Rotation

- **Increasing static non-axisymmetric field leads to bifurcation in torque-balance equilibrium of plasma**
  - Rotation must jump from a high value to essentially locked
- **"Induction motor" model of error field-driven reconnection [Fitzpatrick]:**
  - Plasma rotation at critical point, V_{crit}~1/2 of unperturbed rotation, V_0
- **Lower neutral beam torque gives lower V_0, therefore a lower V_{crit} at entrance to forbidden band of rotation**

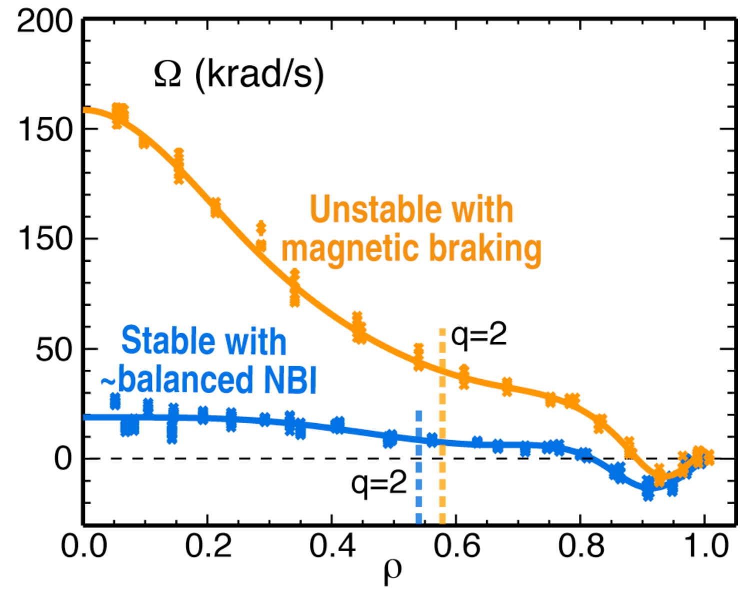
Magnetic braking thresholds



With Optimal Error Field Correction, RWM Stabilization at Very Slow Plasma Rotation Sustained for >300 Wall Times

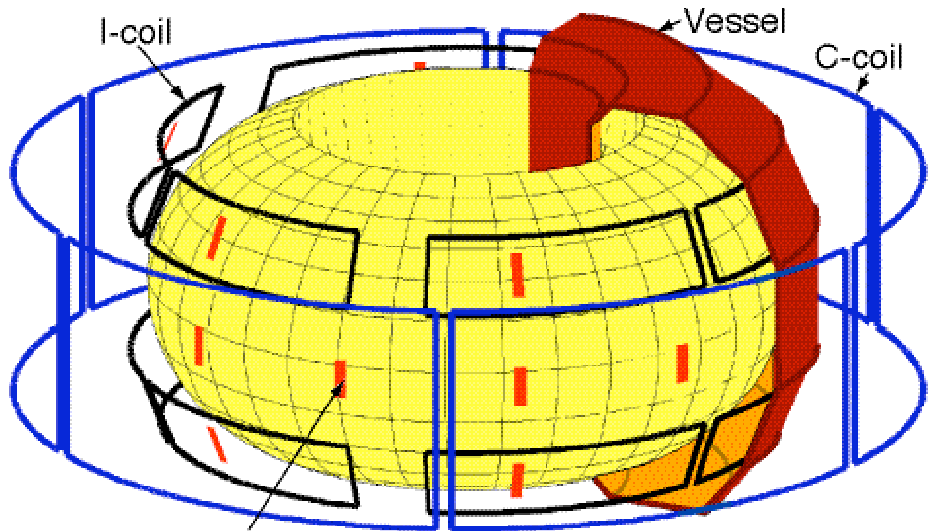


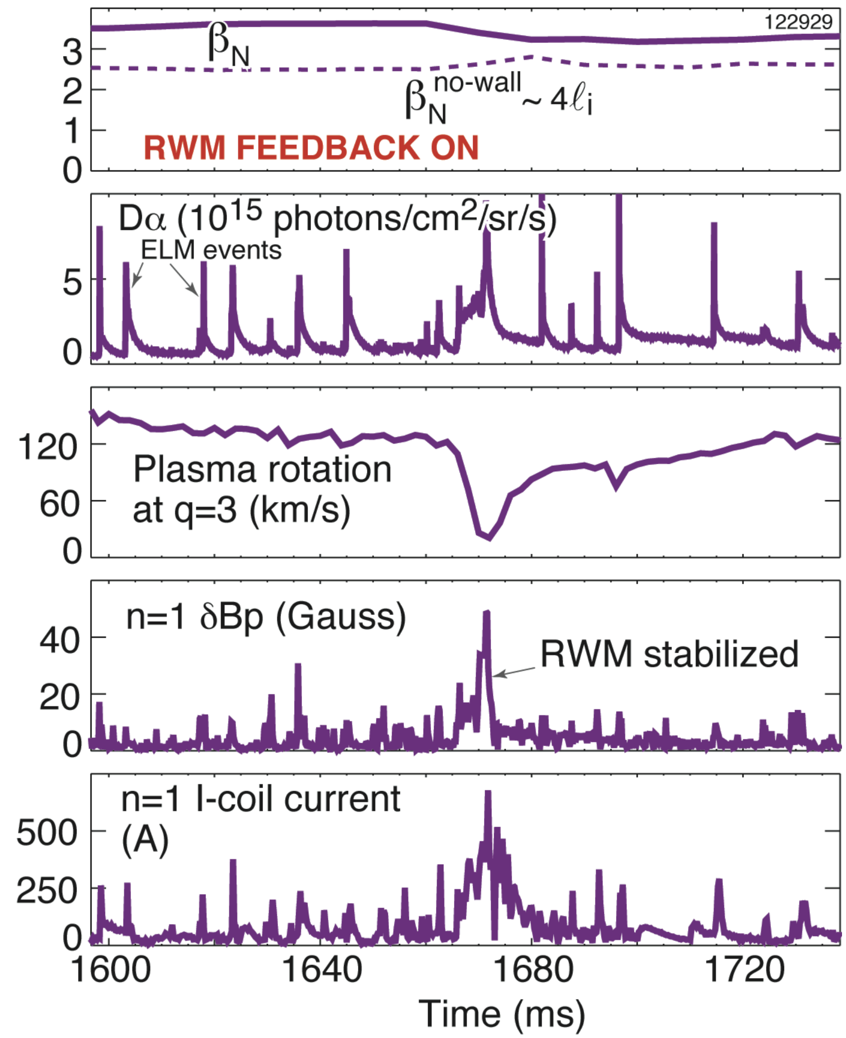
- Plasma rotation just above Ω_{crit} ~0.35% Ω_A at q=2 is sufficient to sustain β_N, above no-wall limit



In High Performance Plasmas, Active RWM Feedback May Still be Required

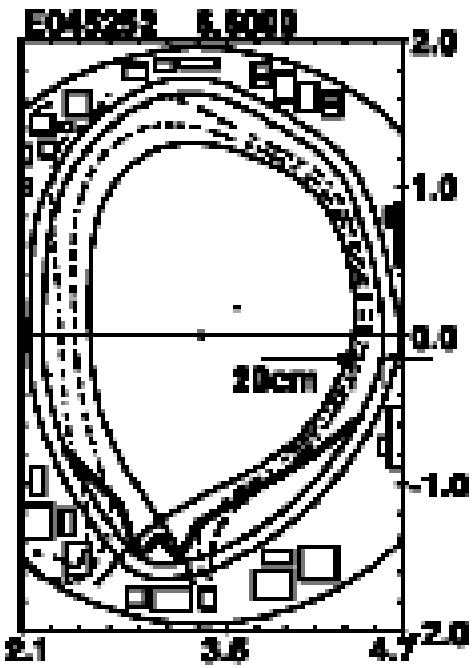
- **In DIII-D, large, slow-varying n=1 currents in external coils provide error field correction, maintain high rotation**
- **Large ELMs can lead to loss of rotational stabilization**
- **Smaller, faster-varying n=1 currents in internal coils can respond to transient events, maintaining RWM stabilization**

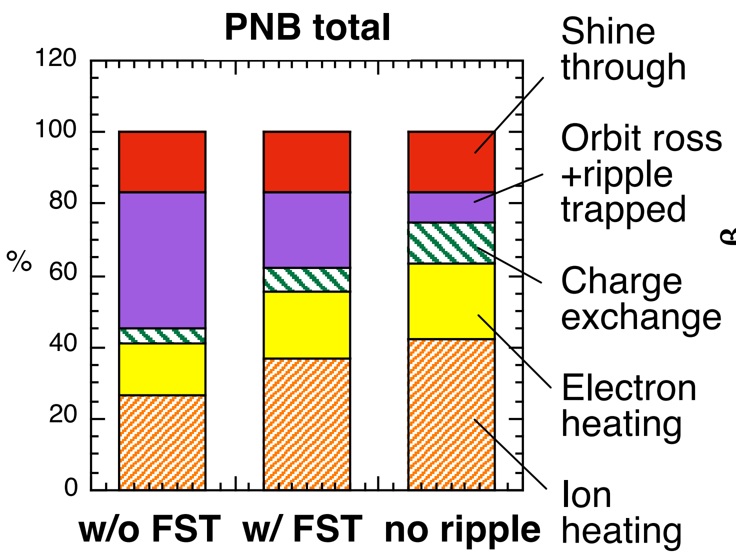


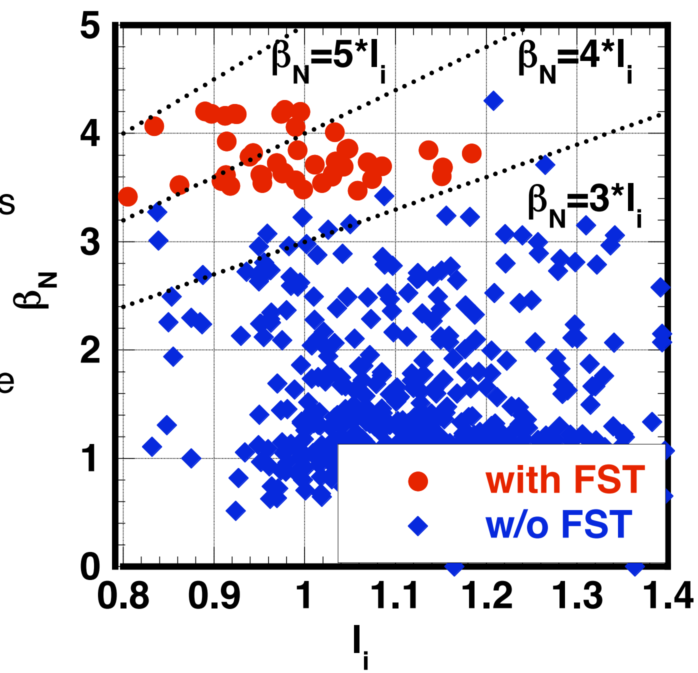


Ferritic Steel Tiles (FST) lead to high beta on large JT-60U plasmas

- **Before installing ferritic steel tiles,** few large plasmas reached the ideal beta limit, however it is difficult to exceed it due to lack of NB power.
- The net NB power with FST is 1.34 times larger than that w/o FST due to reduction of ripple loss.
- Increase net power of ~3.5 MW corresponding to 2 tangential beams.
  - --> Change rotation by one-way tangential NB injection.
- Achieved high β_N~4.2 exceeding ideal limit at l_i<1.2 and V_p>70m^3 (β_N~3.4 w/o FST).

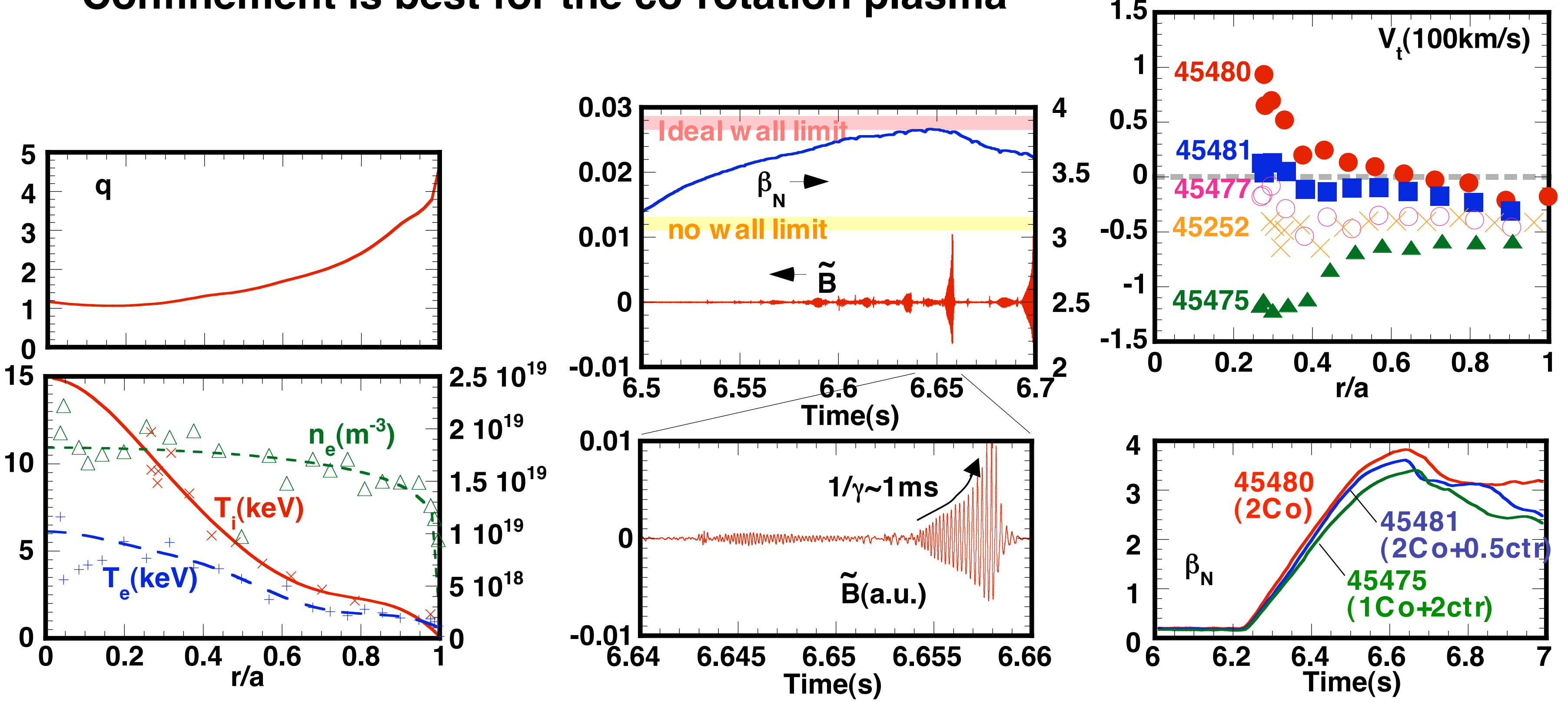






β_N is restricted by the MHD instability

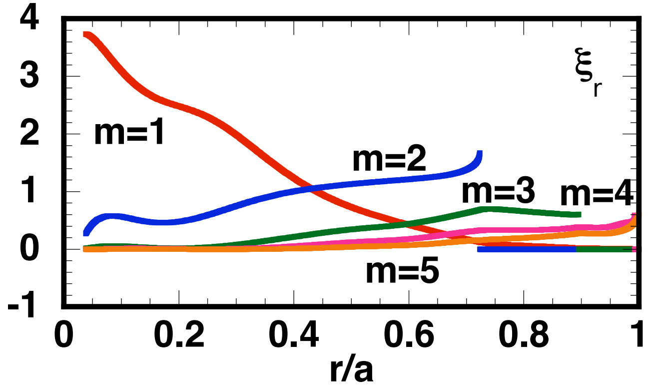
- B_t=1.575, I_p=0.9MA, q_{min}~1.1, q_{95}~3.5,d/a~1.2
- High β_p-H mode plasma (ITB&ETB)
- The n=1 (m~3) mode appears at high beta region.
- The mode grows with growth time 1/γ ~1ms before collapse.
- Frequency of the mode is ~1-5 kHz
- Highest beta is obtained with co-rotation
- Confinement is best for the co-rotation plasma

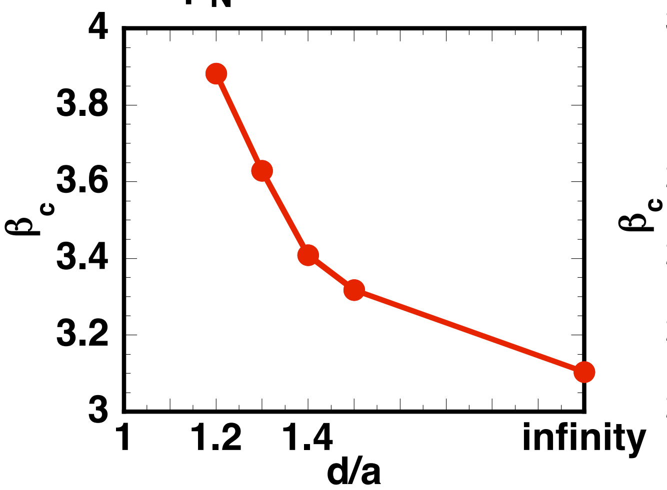


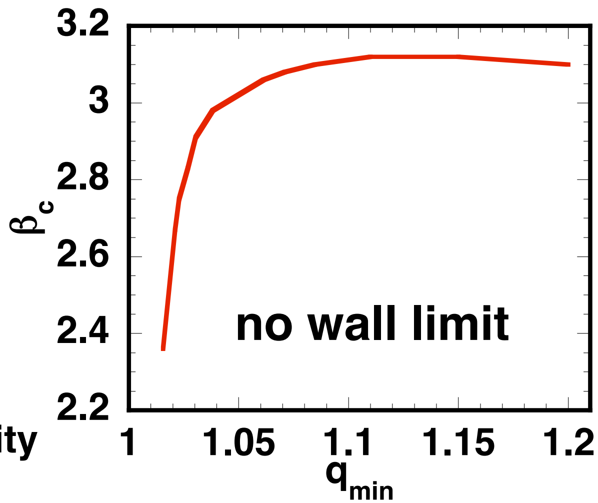
β_N is determined by the ideal wall limit. (MARG2D code)

JT-60U

- The dominant poloidal component is m=1 due to strong ITB at r/a~0.2.
- The mode is stabilized by the wall and ideal wall limit is β_N~3.9 for the plasma at d/a=1.2 when no wall limit is β_N~3.1.
  - -->Beta reaches ideal wall limit
- Current profile is determined by competition between current diffusion and increasing bootstrap current
- Small q_{min}(~1.0) for small and ctr rotation plasmas due to small bootstrap current.
  - --> Critical beta decrease at q_{min}<1.1. (q_{min}~1.08 at highest beta plasma).
  - --> Small ideal wall limit.
- The critical beta is affected by the peripheral plasma current.
  - --> Small current ramp down before NB injection to reduce edge current.
  - -->Achieved highest beta β_N~4.2.

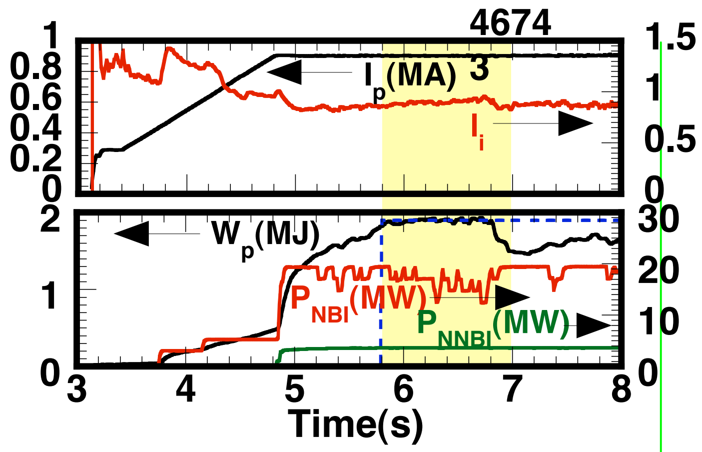


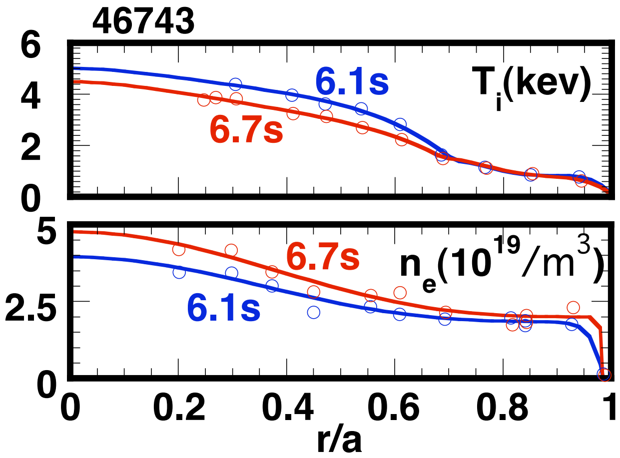


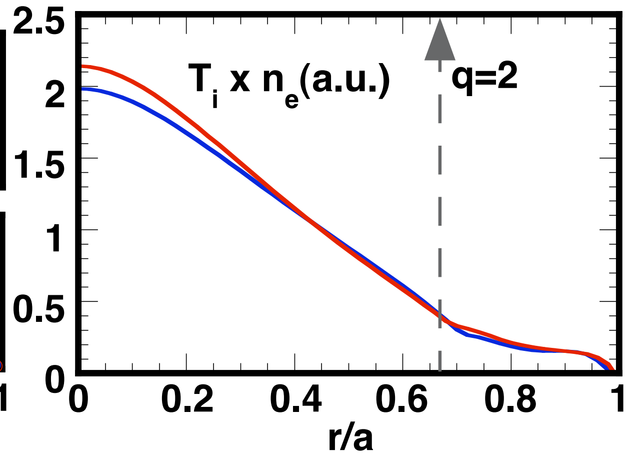


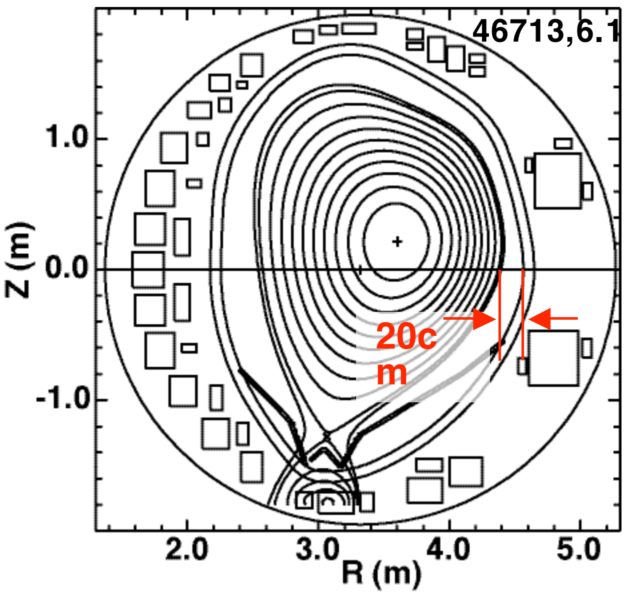
RWM experiment for critical rotation

- B_t=1.575 T, I_p=0.9 MA, q_{min}~1.2, q_{95}~3.5
- d/a~1.2
- To increase q_{min}, pre-NB is injected during current ramp up
- β_N is kept constant and change the tangential NB from ctr-NB to co-NB.
- Pressure and current profile is also kept constant



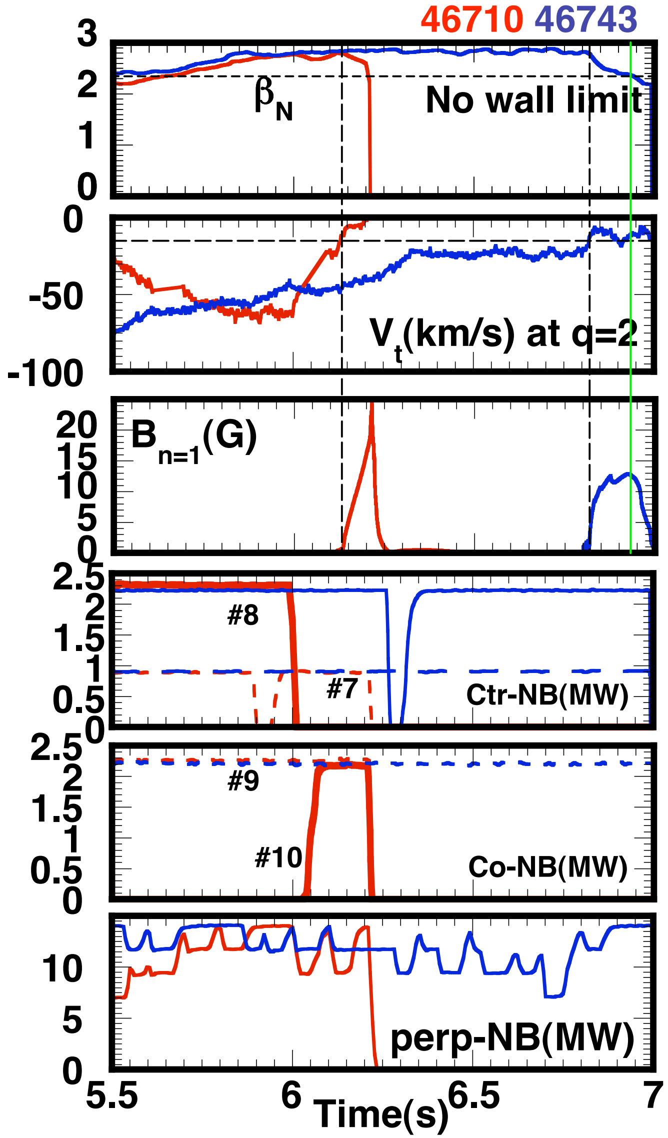


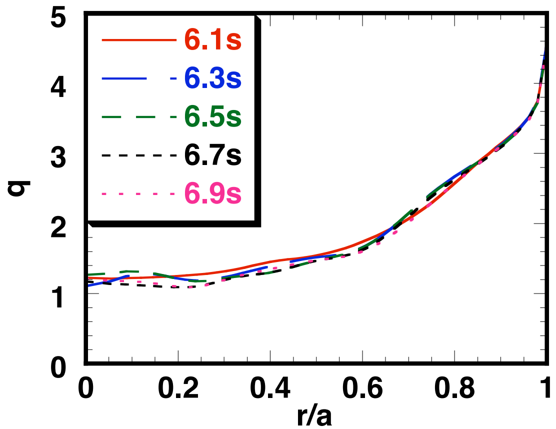


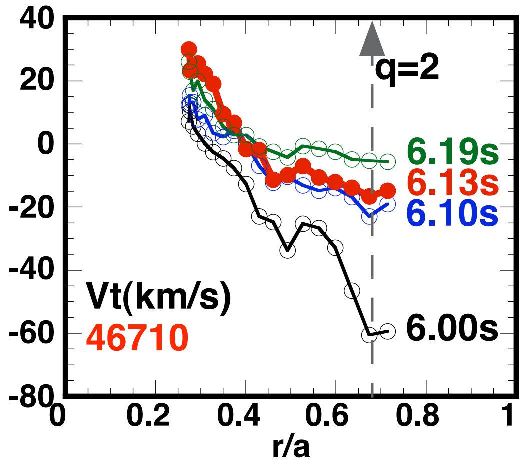


RWM is suppressed by plasma rotation

- β_N is kept constant and change the tangential NB from ctr-NB to co-NB.
- Rotation can be controlled by changing tangential NB combination
- Disruption or collapse occurs at Vt~10 km/s ->n=1 mode grow with $1/\gamma$ ~10 ms .
- The mode suppressed after $\beta_N < \beta_{N\ \text{no-wall limit}}$
- To investigate the effect of beta on critical rotation, we change the constant β_N.

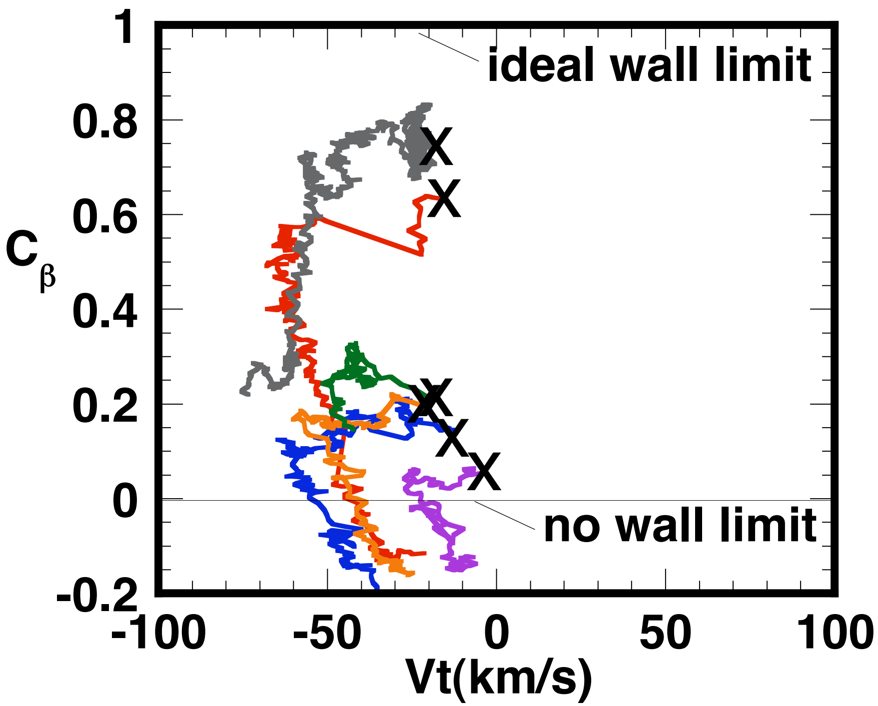


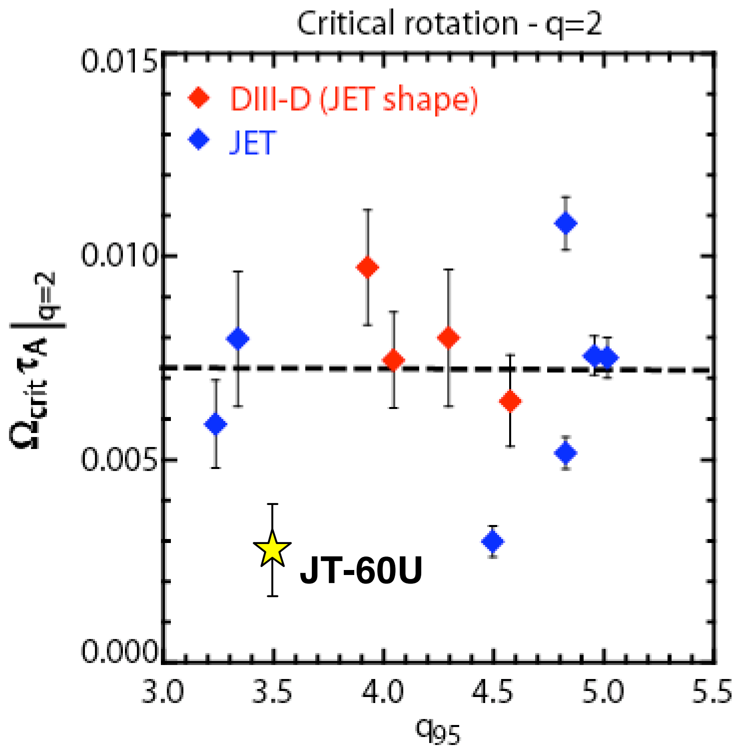




Critical Rotation

- Critical rotation V_c ~5-20km/s
- V_c/V_A~0.3% (q_{95}~3.5) is much smaller than previous DIIID and JET results using magnetic braking
  - Indicating importance of error field?
- V_c does not increase as C_β increase

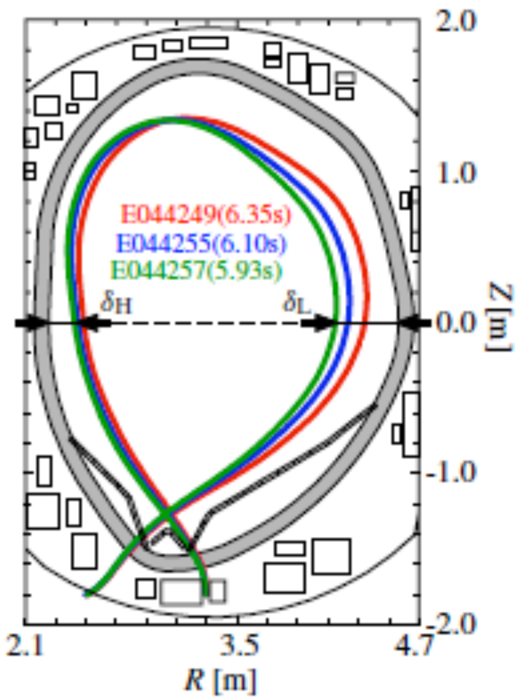


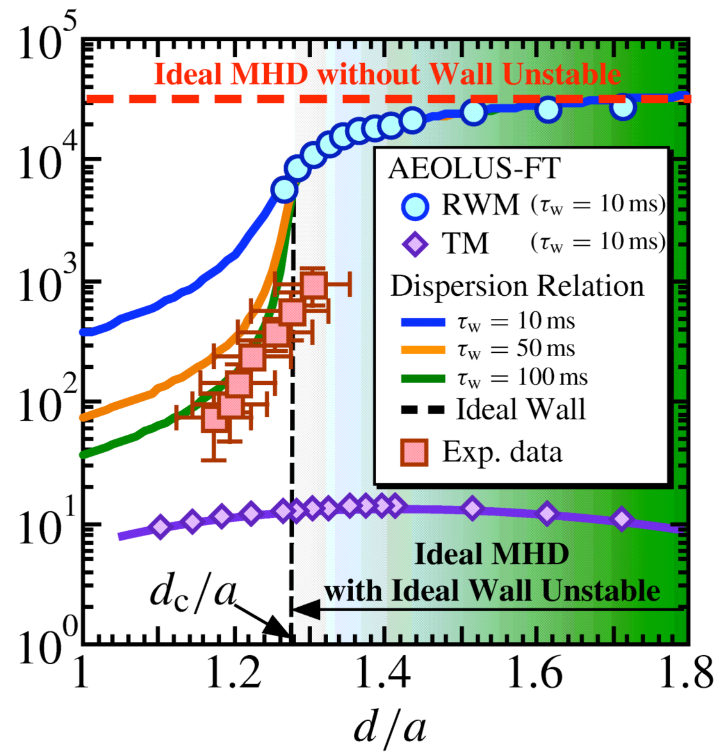


Current driven RWM experiment for wall effect

Experimentally obtained growth rates are consistent with RWM, wall stabilization effects were observed

- **AEOLUS-FT, which can take into account the resistivity of the wall, found 3/1 kink and 2/1 tearing branches.**
- **The above dispersion with no plasma rotation and no dissipation is consistent with AEOLUS-FT.**
- **These modes have been observed in the region where the ideal MHD mode with ideal wall is stable.**
- **From the strong dependence, the observed modes can be identified as RWM.**





New Hardware Capabilities Allow Simultaneous Discovery of Low RWM Rotation Threshold in DIII–D and JT-60U

- **The plasma rotation needed for RWM stabilization is much slower than previously thought**
  - Achieved with neutral beam line re-orientation in DIII-D:
    - Balanced neutral beam injection -> lower injected torque and plasma rotation with minimized non-axisymmetric fields
  - Achieved with ferritic steel tiles in JT-60U:
    - Reduced ripple loss -> higher confinement and β with smaller plasma-wall separation
  - Such a slow rotation should be achievable in ITER
- **Ideal MHD with dissipation (MARS-F with kinetic model) in agreement with new low threshold rotation for RWM stabilization**
- **Even with sufficient rotation, active feedback may still be needed, but system requirements could be reduced**
- **RWM stabilization allows demonstration of high performance tokamak regimes (β_N>4)**

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