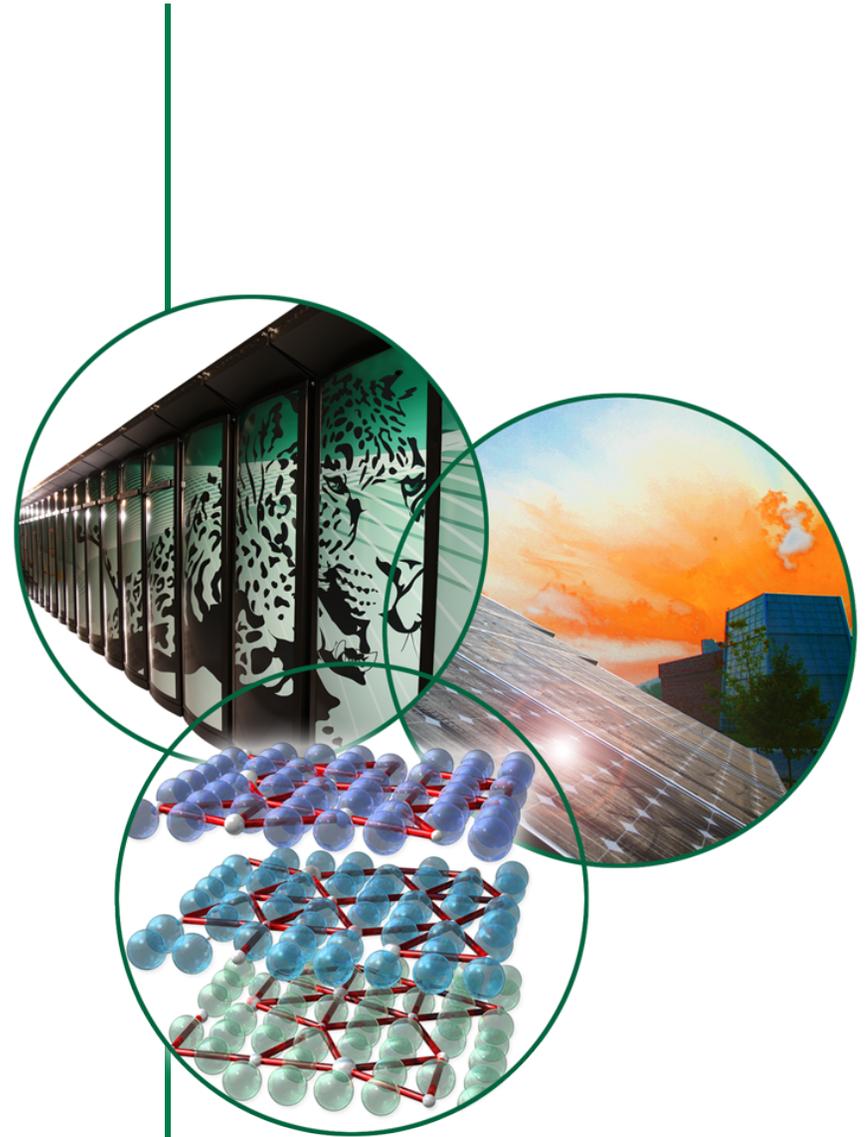
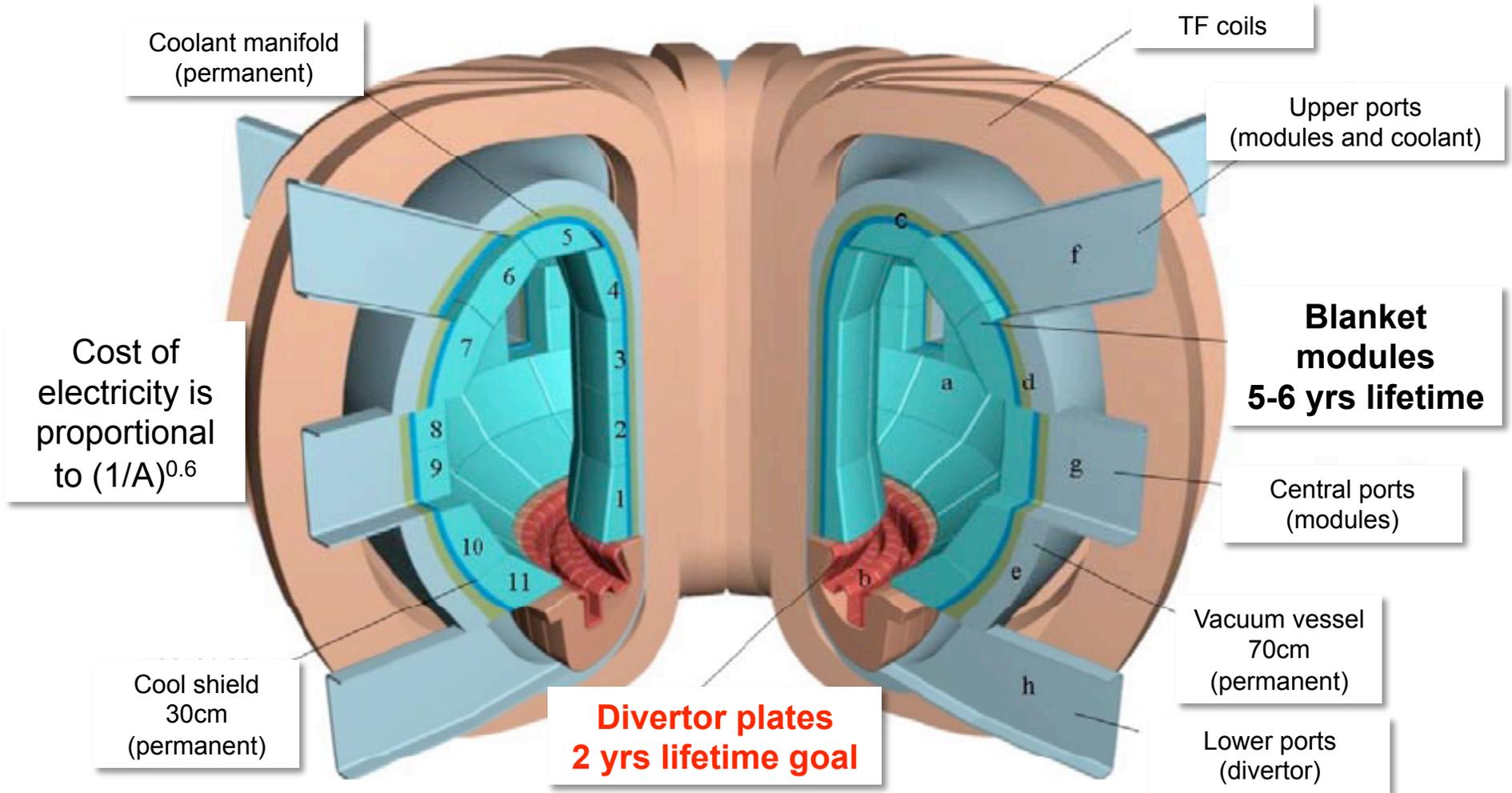


Addressing PMI Science and PFC technology for ITER, FNSF and DEMO

Juergen Rapp



Lifetime of divertor will determine availability of reactor



Main driver of scheduled maintenance: divertor (and blanket)

Challenge: thermal loads



Re-entry vehicle



Space Shuttle rocket nozzle

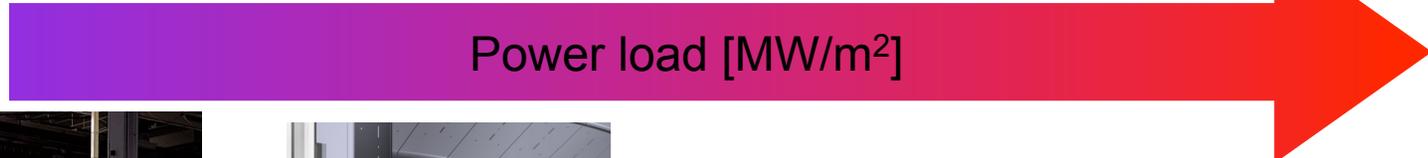
PWR

~1

<10

85

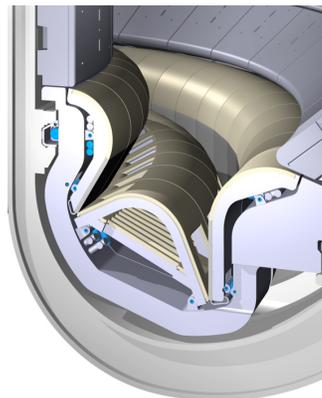
2000



Power load [MW/m²]



GE90-115B



ITER steady-state

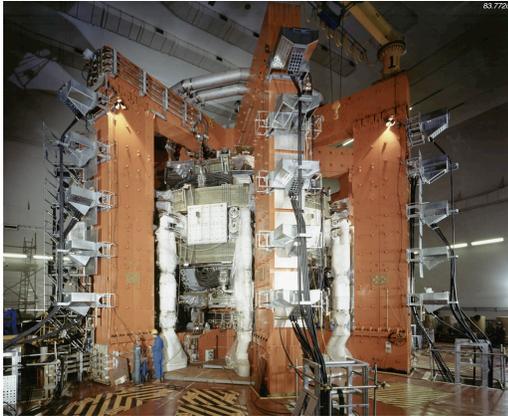
Outer divertor:
1200 C
Inner divertor:
800 C



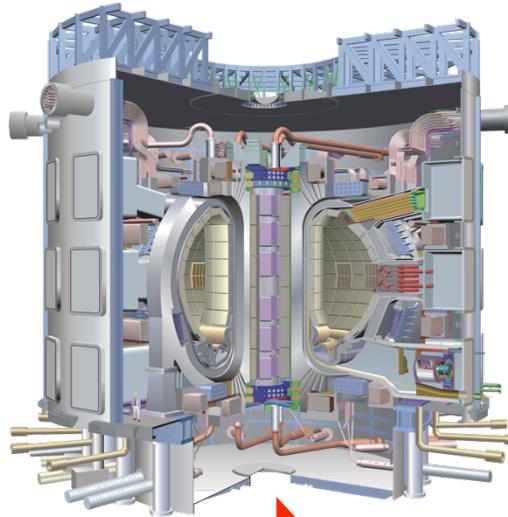
ITER transients
(1ms lifetime)

Challenges for PFCs: fluxes and fluence

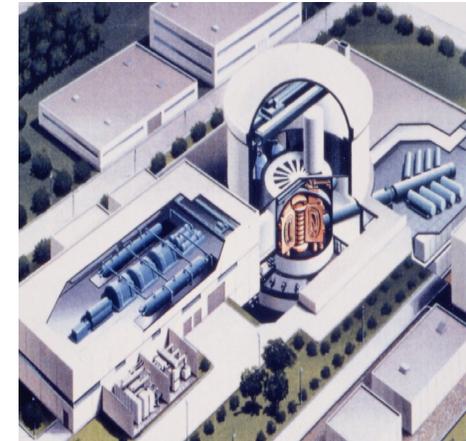
JET



ITER

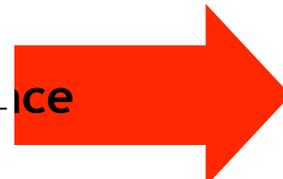


Fusion Reactor



Material circulation due to gross erosion, rough estimations
 50 times higher ion fluxes

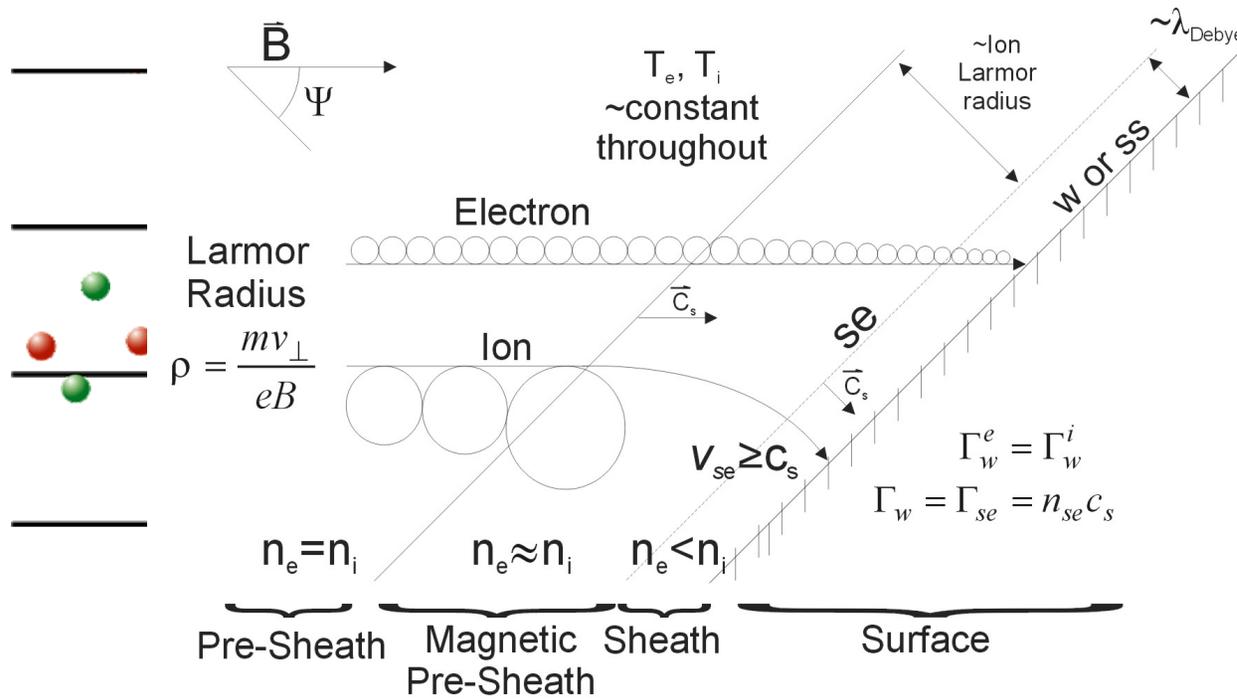
Worst case erosion rate ~ m/yr



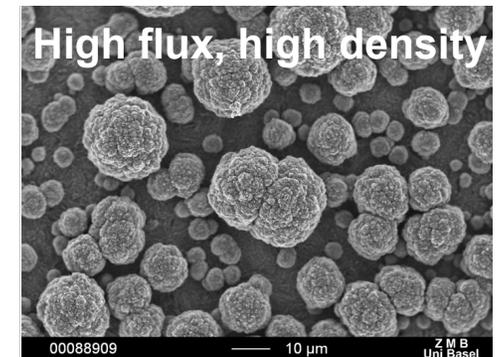
Stangeby, AW Leonard, Nucl. Fusion 51 (2011) 063001

| Device | P_{heat} (MW) | Annual run time (s/year) | Fluence of D/T ions to surfaces (TC/yr) ^a | Beryllium circulation rate (kg/yr) | Boron circulation rate (kg/yr) | Carbon circulation rate (kg/yr) | Tungsten circulation rate (kg/yr) |
|---------|------------------------|--------------------------|--|------------------------------------|--------------------------------|---------------------------------|-----------------------------------|
| DIII-D | 20 | 10^4 | 0.0007 | 2.8 | 1 | 0.5 | 0.7 |
| JT-60SA | 34 | 10^4 | 0.0017 | 4.8 | 1.7 | 0.9 | 1.2 |
| EAST | 24 | 10^5 | 0.01 | 34 | 11 | 5 | 8 |
| ITER | 100 | 10^6 | 0.4 | 1680 | 580 | 270 | 410 |
| FDL | 100 | 10^7 | 3 | 13 400 | 4600 | 2100 | 3300 |
| Reactor | 400 | 2.5×10^7 | 40 | 141 000 | 50 000 | 22 000 | 31 000 |

Plasma Surface Interactions



Re-deposition
Co-deposition of
hydrogen
ion
chemical and physical
sputtering
ion
implantation
etching (metals)



Strongly Coupled PSI regime:

- 1) Eroded material is trapped in plasma (highly collisional)
- 2) High fluence \Rightarrow thick layers of re-deposited material

Every surface atom is displaced $\sim 10^7$ times in a divertor lifetime

- Material in a reactor divertor is NOT what was installed, we need a way to create and test plasma-reformed surfaces

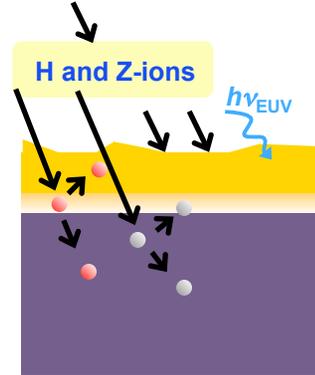
Plasma Surface Interactions

Complex systems with many species and layers

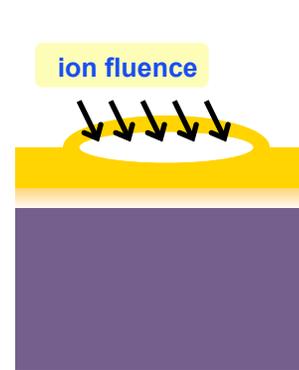
H, D ions plus low-Z and high-Z ions:

He, C, (Be), W, N, Ne, Ar

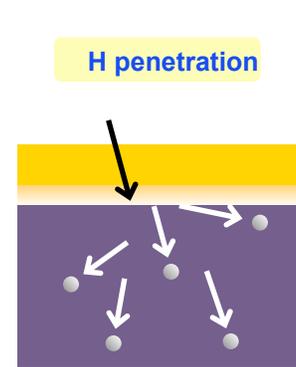
erosion/ intermixing & ion implantation



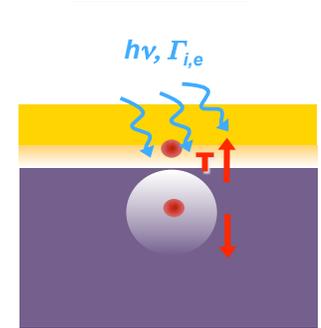
blistering & flaking



H bombardment & retention



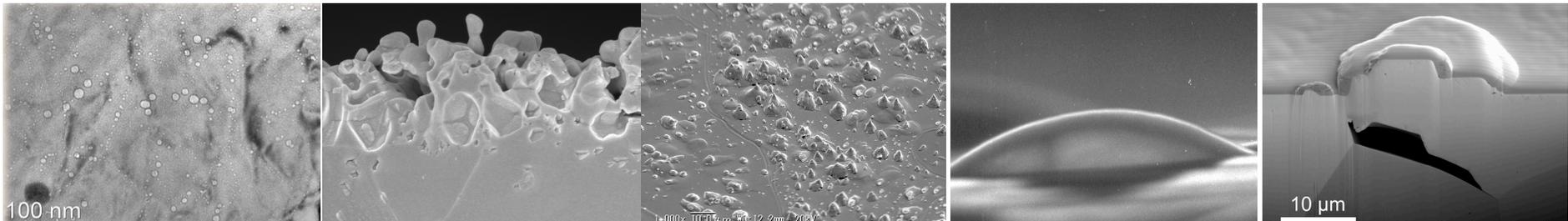
diffusion & permeation, annihilation



Extreme conditions change materials considerably

Irradiation by neutrons and helium will enhance surface modification !

Note: At the end of the PFC lifetime the surface has moved through the bulk material



Void formation, 9 dpa

Bubbles in W by He

Blisters within grains

Large Blisters due to voids at grain boundary

Value of linear plasma devices, when compared to toroidal devices

- Good diagnostic access
- Diagnostics are readily exchanged (maintenance, upgrades)
- Surface diagnosis possible without breaking vacuum
- High flux ($10^{24} \text{ m}^{-2}\text{s}^{-1}$), high fluence discharges possible (accelerated lifetime tests), like that of ITER, FNSF, DEMO
- Well controlled continuous plasma conditions vs. tokamaks enduring different conditions in short durations

Inter-ELM, ELM, L-mode phase, ramp-up, ramp-down in different operation scenarios for many campaigns, venting.....

- Research objectives of tokamaks and stellarators rely on success of PSI, yet only a fraction of the time is allocated to active PSI research
- Damage and lifetime studies of PFCs in tokamaks, stellarators are mostly not allowed or discouraged due to excessive risks and expenses (change of wall components; shutdown times) and issues of reactor relevance
- Device and operation costs of linear plasma generators are small fractions than that of tokamaks and stellarators

➤ **Linear plasma devices can complement power exhaust science carried out on toroidal devices in a synergistic way.**

Need for Upgraded or New Linear Plasma Facility

- Upgraded capabilities should include but are not limited to:
 - Access to high density low temperature reactor divertor plasma conditions ($n_e > 10^{21} \text{ m}^{-3}$, $T_e \sim 1 \text{ eV}$)
 - Parallel power fluxes of up to 40 MW/m^2
 - Ion fluxes of more than $10^{24} \text{ m}^{-2}\text{s}^{-1}$
 - Steady-state conditions for the above mentioned parameters (-> high fluence)
 - Capability to expose irradiated and toxic material samples
 - Ability to control electron and ion temperature separately
- Requirement: high density plasma source with sufficient power based on RF-technology
- Anticipated costs for such an Upgrade or New device are $< \$15\text{M}$.

DOE acknowledges importance of Plasma Material Interaction (PMI) in fusion

DOE ReNeW (Research Needs Workshop) identified

First wall materials and compatibility with fusion reactor relevant plasmas (theme 3)

to be addressed for rapid and efficient realization of fusion energy i.e. **Thrust 10**:

Decode and advance the science and technology of plasma-surface interactions

DOE FESAC panel (Fusion Energy Science Advisory Comm.) recently identified the need of an

Upgrade and/or New Build of linear plasma test stands with medium scale facilities

| Requirement | Thrusts |
|---|-------------------------|
| Diagnostic investment for edge characterization | 1 and 9 |
| Dedicated experimental time for edge characterization | 1, 5, 9, 10 |
| Improved models and code components for edge region | 9 and 10 |
| Innovative divertor concepts and testing | 9 and 11 |
| Transient impact on plasma facing components | 2, 6 and 10 |
| Innovative design of solid surface PFCs | 10 and 11 |
| Testing necessary to validate codes, improved physics models, and new designs | 10 and 11 |
| Liquid surface development | 11 |
| Improved diagnostic parts in edge | 1, 2, 9, 10 , 11 |
| Antenna and launcher development | 10 |
| Internal coils | 2 and 5 |
| Integrated demonstration of taming plasma material interactions | 12 |