

*US Strategies
for an Innovative
Stellarator-Based FNSF*

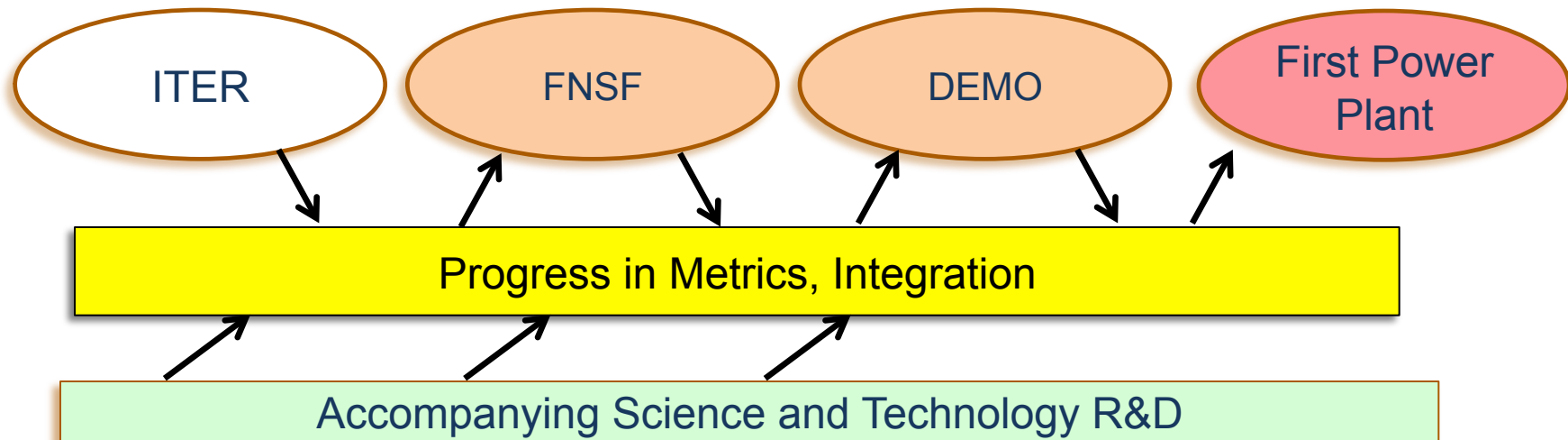
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For the US Stellarator Steering Committee

FESAC Strategic Planning Panel

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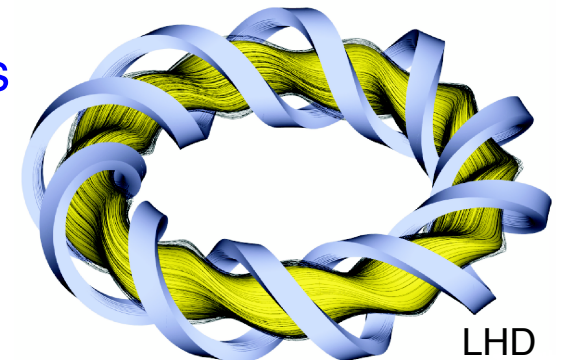
Context



FNSF: fully integrated fusion plasma and technological environment

- Long-pulse, high duty factor
- Integration testing and validation; tritium breeding

Stellarators: helical magnetic field torus, like tokamaks
full field from 3D coils, 3D plasma shaping



LHD

Stellarators Already Provide Advanced Characteristics

Steady-state: field from 3D coils, not plasma current

- ✓ No disruptions.
- ✓ No current drive \Rightarrow potential high fusion gain, higher reliability
- ✓ Quiescent high-beta $\geq 5\%$,
- ✓ Energy confinement similar to tokamaks.
- ✓ Very high density limit \Rightarrow potential higher fusion reactivity
colder edge, for easier divertor
reduced fast-ion instability drive
- ✓ No need for feedback stabilization \Rightarrow simplify plasma control,
reduce diagnostics needed in fusion environment

Closes some technical gaps, reduces some R&D needs.

Simplifies FNSF and DEMO designs.

Need to demonstrate these capabilities can be simultaneous

Stellarator Research is Active World-wide

Large international programs:

- LHD, R=3.7m superconducting, partially optimized (Japan, 1998)
- W-7X, R=5.5m superconducting, quasi-omnigenous (Germany, 2015)

US:

- Historically, strong theory program.
 - Methods to optimize confinement in 3D
 - quasi-symmetry (QS): tokamak-like transport
- Pioneering novel Concept Exploration experiments, e.g.
 - HSX, quasi-helically symmetric
 - CTH, disruption onset thresholds
- NCSX project: partially built mid-scale experiment
 - study sustainable high performance, quasi-axisymmetric
 - synergy: tokamak-like transport and stellarator stability.
 - cancelled in 2008, due to cost-overruns. Major components done.

QS Stellarator FNSF has Moderate Size

	R (m)	$\langle a \rangle$ (m)	B (T)	β (%)	Pfus (MW)	Neut. Wall load. (MW/ m ²)
<ul style="list-style-type: none"> W7-X-like H.Wobig et al, NF 43 (2003) 889 	18	2.1	4.5	3.6	1500	0.9
<ul style="list-style-type: none"> ARIES-CS-like J.Menard et al, NF 51 (2011) 103014 	4.75	1.05	5.6	6	529	2
<ul style="list-style-type: none"> LHD-like A. Sugara et al, FED 87 (2012) 594 	14.4	2.5	4.7	5	3000	1.5

All target FNSF neutron flux. All are ignited or very close to ignition.

Similar to power plant designs -> prototype reactor integration issues directly

All are high gain. Would produce net power: can be pilot plants.

Lack of Current Drive Has Practical Benefits

- Minimizes wall penetrations & blocking of breeding blankets
- Relieves engineering constraints. Provides design margin on performance. Makes design easier.
- Simplifies sub-systems and control
- Strongly reduces recirculating power. Allows net power production at lower fusion power.

For stellarator DT experiments:

Component Test Facility = High Q_{DT} = Pilot-plant

Stellarator R&D Gaps to FNSF

- Integrated high performance of QS-optimized stellarators
 - Simultaneous high beta, high confinement, without disruptions. Benign ELMs.
 - Requires experimental validation
- Simplified coil design, via new coil strategies or simpler shape
- Predictive capability for plasma behavior and operation
 - Operating limits
 - Validation of theoretical models; including relationship to tokamak physics understanding.
- Effective divertor design, compatible with high performance
- Impurity and fusion ash accumulation control

US Stellarator Initiative is Needed (1)

1. Strong international collaboration with W7X, LHD (G.Wurden)
 - W-7X will be the first large, fully-optimized 3D experiment
 - Long-pulse, high-power, high-beta capabilities. Divertor program.
 - But, not quasi-symmetric – hard to connect with ITER.
 - Project to very large FNSF or DEMO.
2. US mid-scale QS experiment (J. Harris)
 - Integrated high performance for QS; divertor; impurity accumulation; predictive understanding
 - QUASAR using NCSX components (~4 years)
 - Theory-based design for this mission
 - Managed as National & International collaboration.

US Stellarator Initiative is Needed (2)

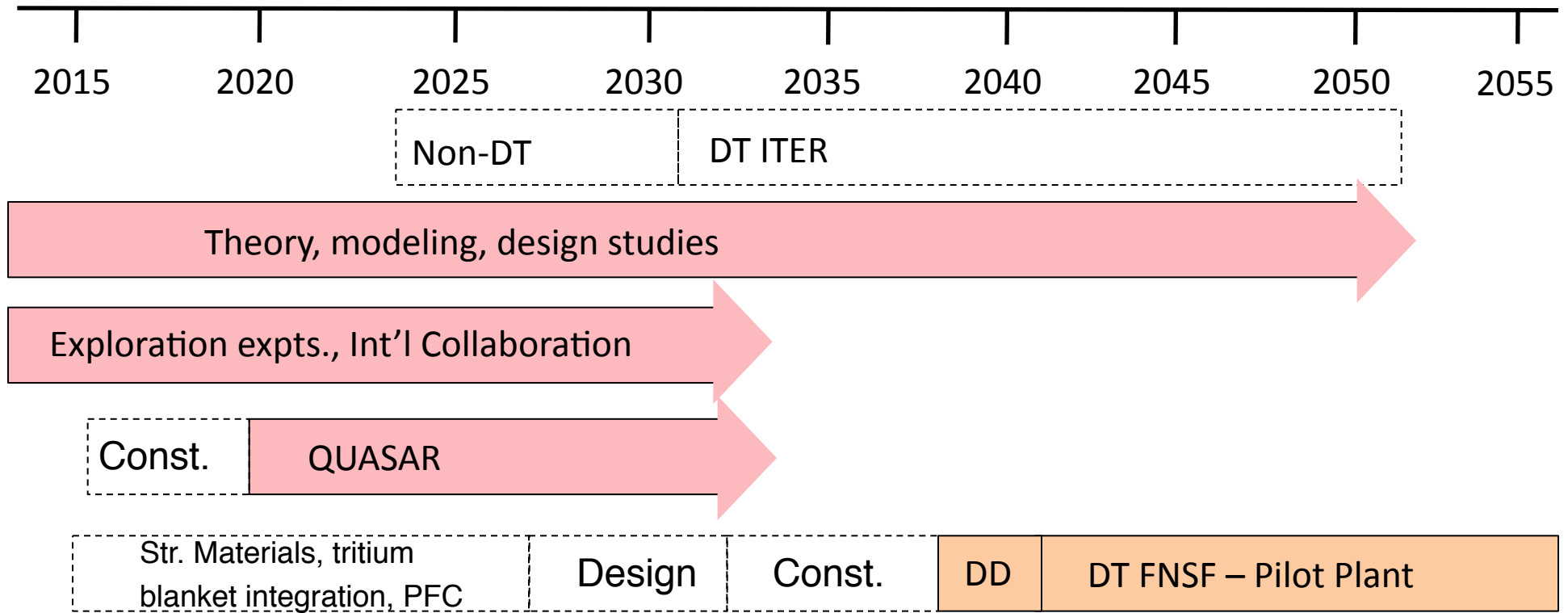
3. Targetted exploration experiments (O. Schmitz)

- Divertor design development
- Simpler coils
- Tests of turbulence optimization

4. Strengthened theory & computation program (M. Landreman)

- Predictive understanding and modeling
- Configuration optimization & improvement
- Simpler coil design
- Design of next-step experiments

Stellarator Initiative Fits in ITER Timescale



- Major reviews of Initiative in
 - ~2015 (start)
 - ~2024 (progress)
 - ~2029 (readiness for next steps, decision on approaches)

Summary

- Stellarators can be a game-changer. Provide many of the needed characteristics for a FNSF and an advanced DEMO.
 - No disruptions. No current drive. High beta.
 - Stellarator FNSF is high gain, can be a pilot-plant.
- US opportunity to lead QS-optimization strategy
 - close connection to tokamak understanding; ITER results
 - can result in similar system scale as tokamaks.
- Need US Initiative to close remaining gaps
 - Strong collaboration with large international facilities
 - Mid-scale QS experiment: integrated performance
 - Concept exploration experiments on specific topics
 - Strong theory and modeling program.