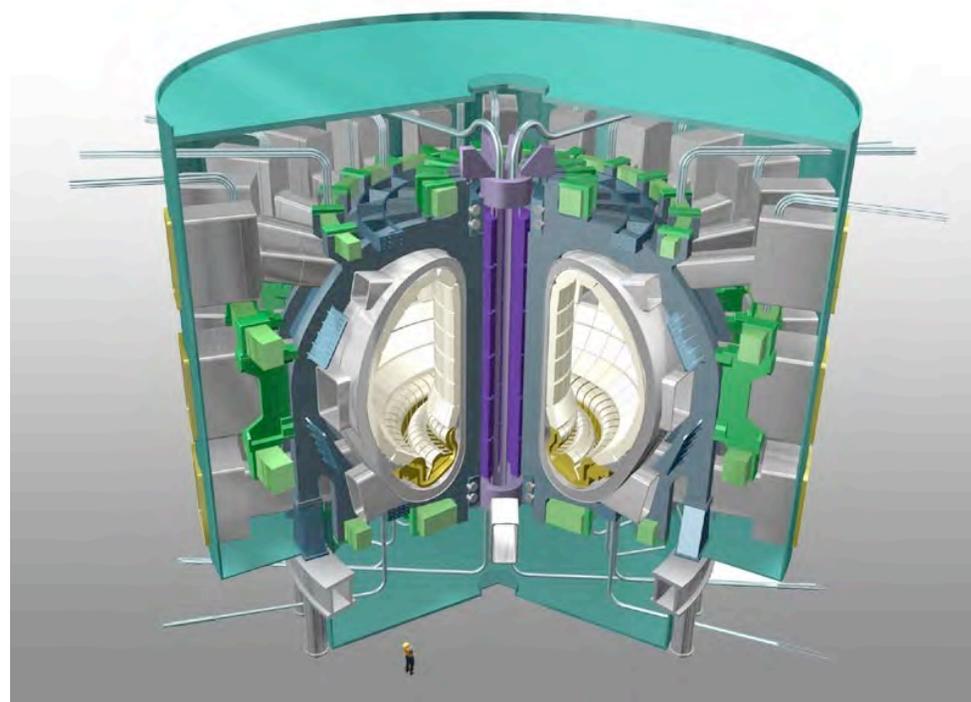


# US ITER Project

*Providing a Facility for  
Burning Plasma Research*

*Working with the US community  
to position the US  
for Burning Plasma Research*



**Ned Sauthoff**

**Project Manager, US ITER Project**

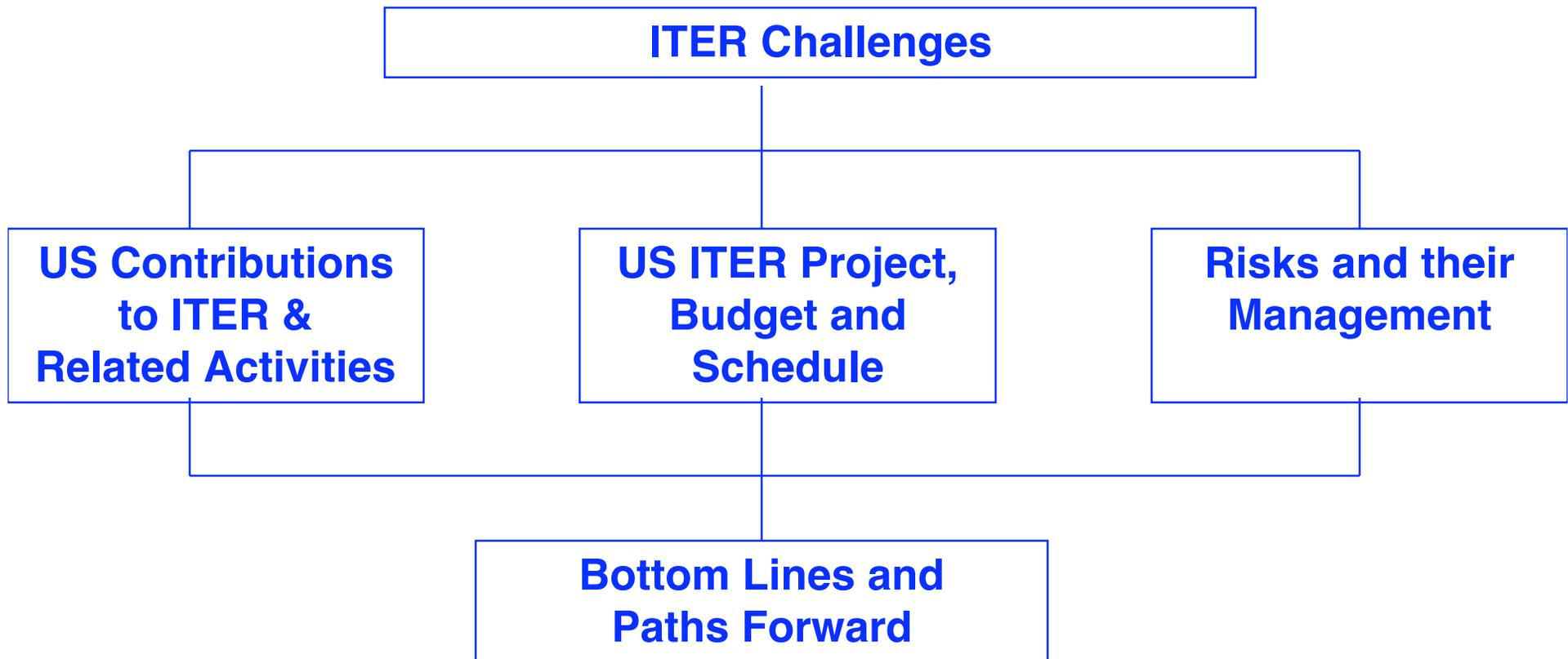
**APS/DPP meeting**

**Philadelphia, PA**

**October 31, 2006**

# Structure of the Talk...

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# Structure of the Talk...

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**ITER Challenges**

**US Contributions  
to ITER &  
Related Activities**

**ITER is a unique and complex facility and program**

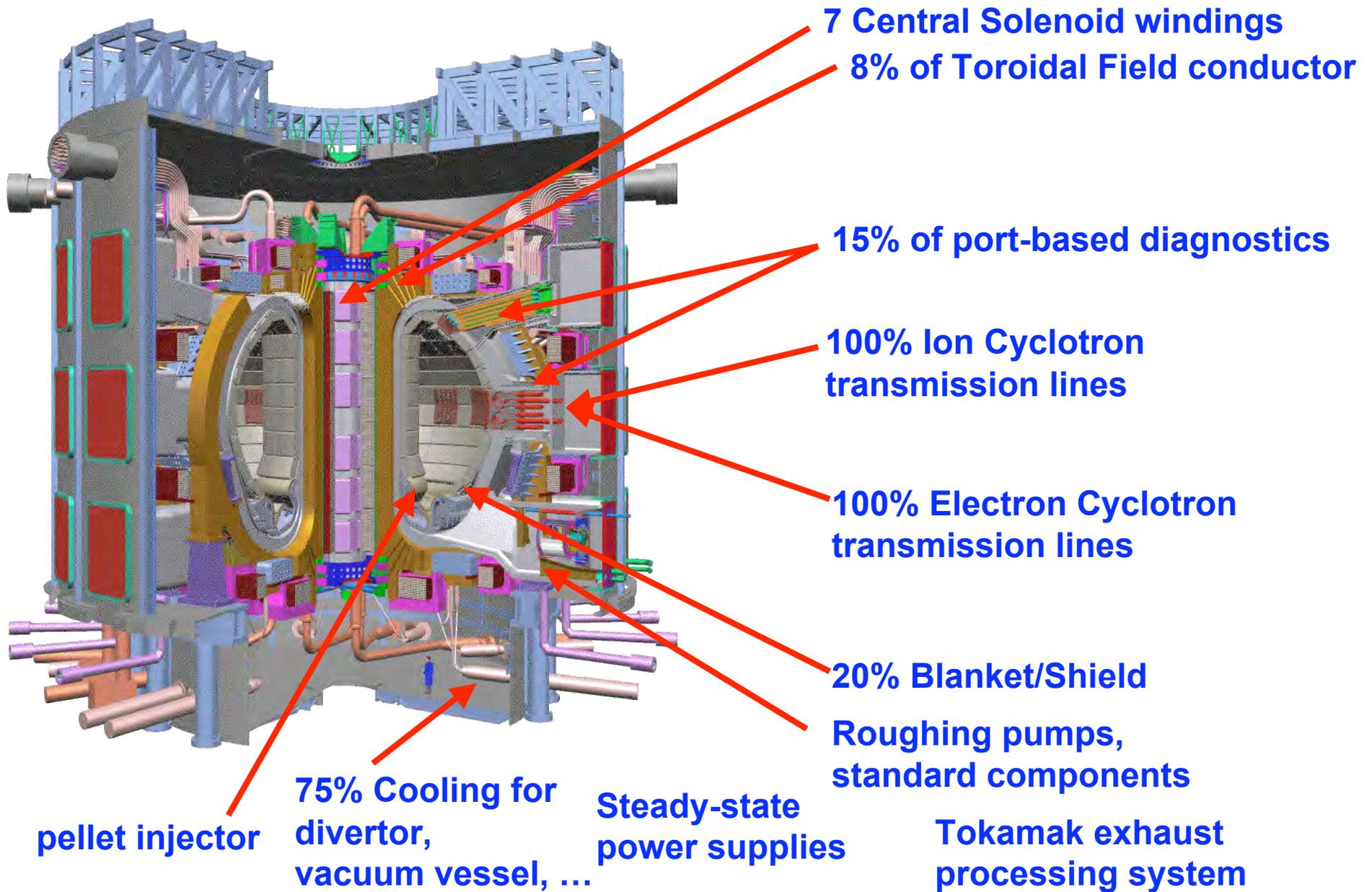
**ITER is also a challenging experiment  
in international collaborative project management**

# Structure of the Talk...

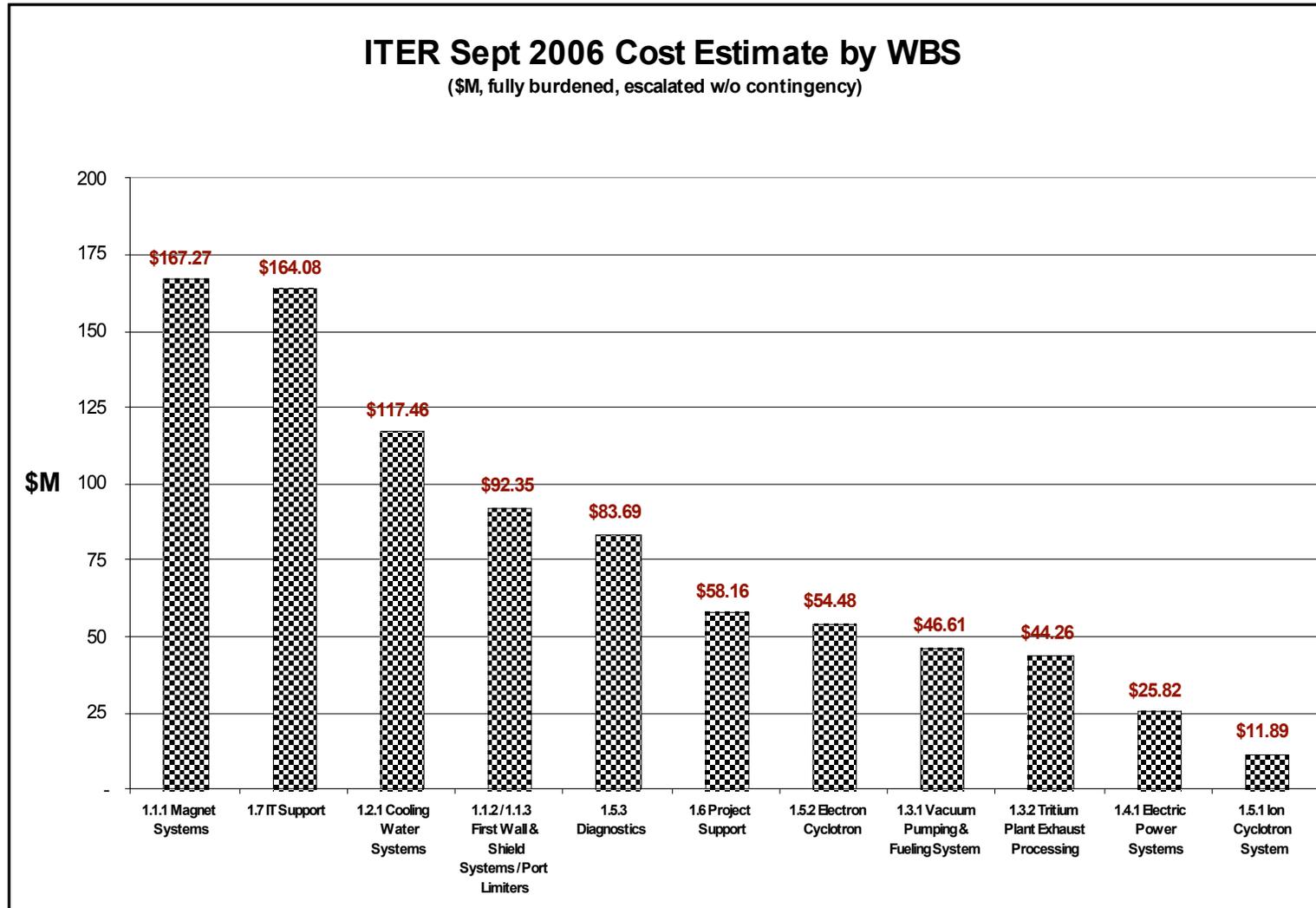
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# 2006 U.S. “in-kind contribution” hardware scopes



# Major Areas of Cost



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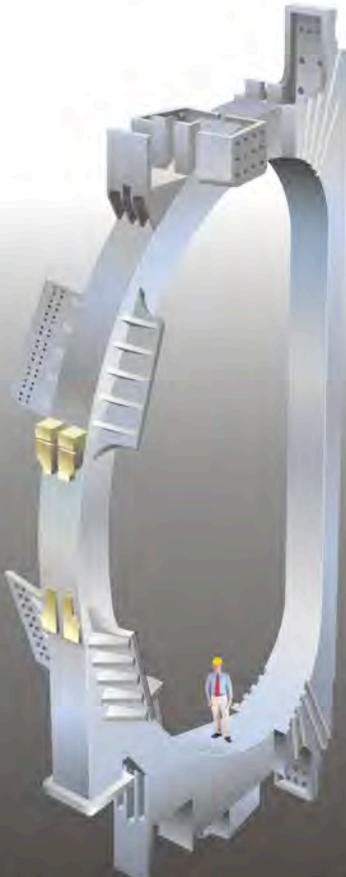
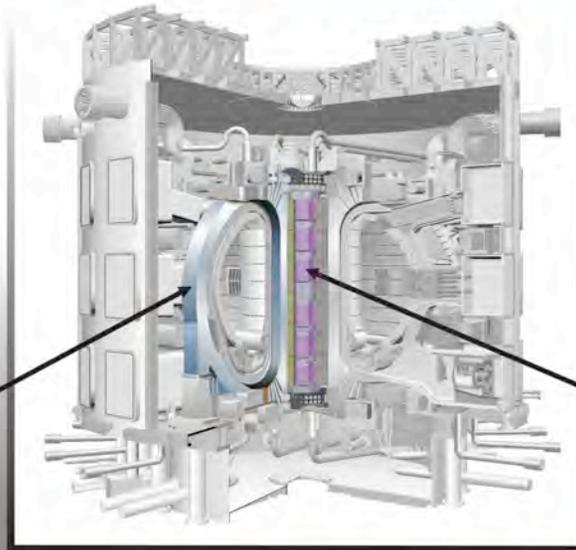
# Magnets

# US Magnet Scope



Toroidal Field (TF) Coils

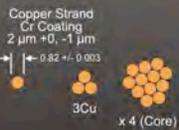
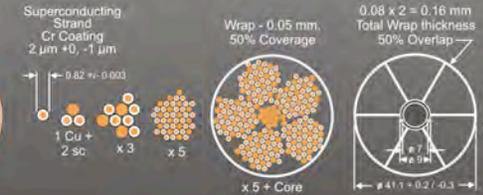
Central Solenoid (CS)



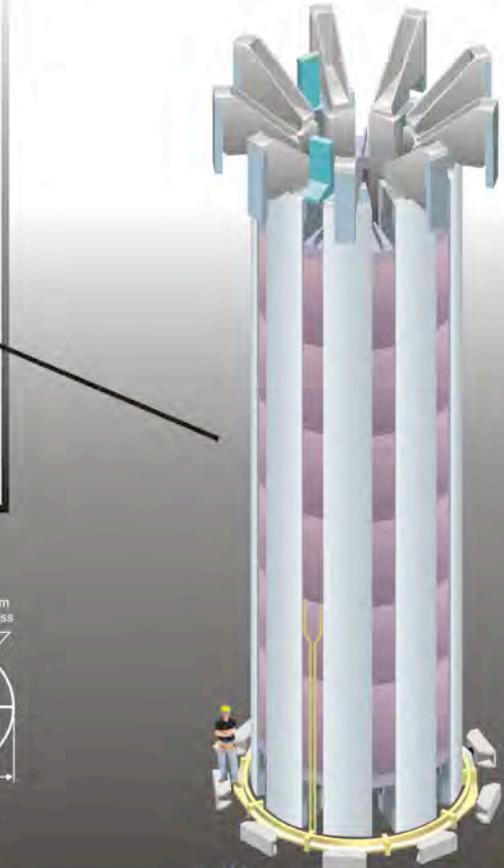
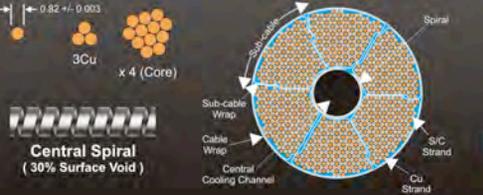
Six of the seven participant teams will provide TF conductor. The US team will fabricate nearly 8 km of TF conductor, including active, dummy, and test samples for qualification.



### Cable Configuration



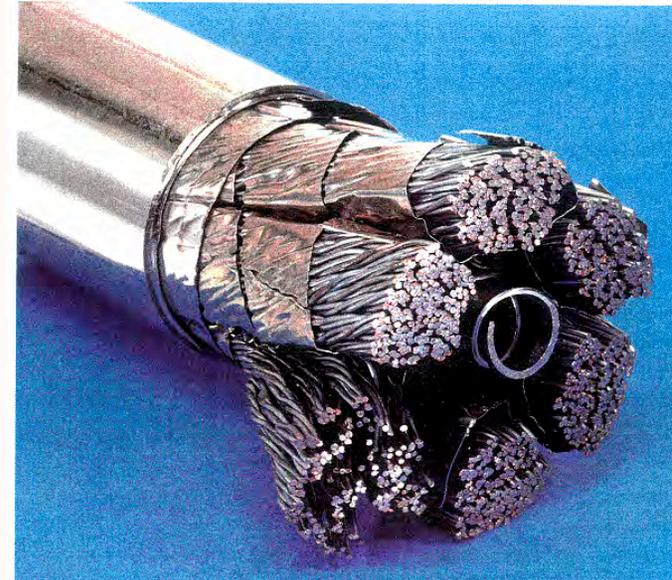
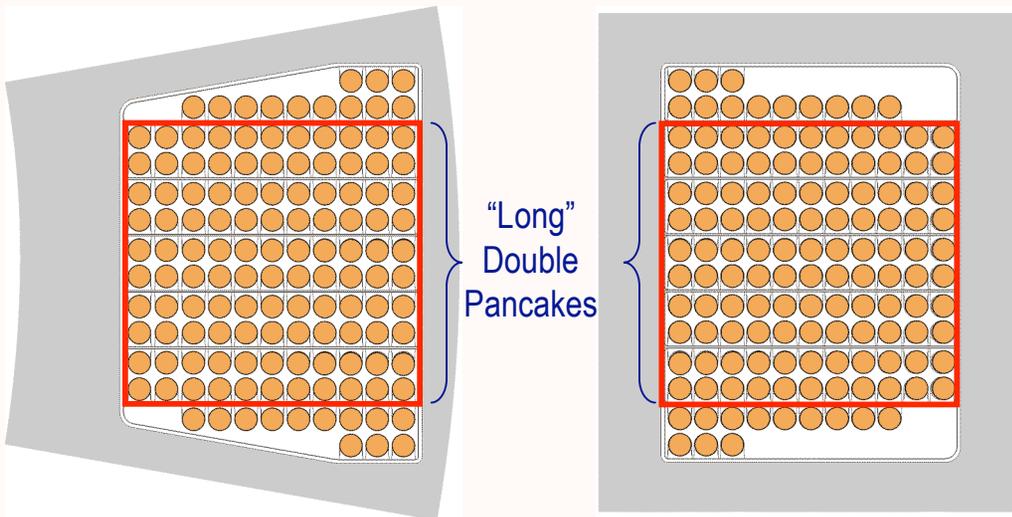
### Cable Cross-Section



The U.S. Team will use CS conductor provided by the Japanese Team



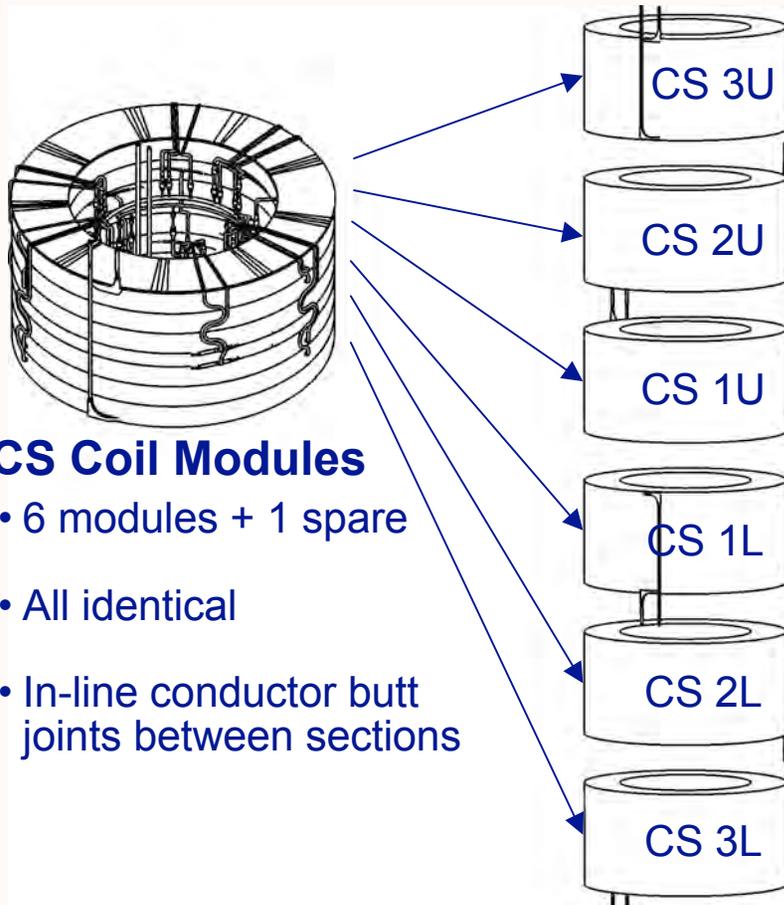
# TF Conductor Production



US contributes about 8% of TF conductor

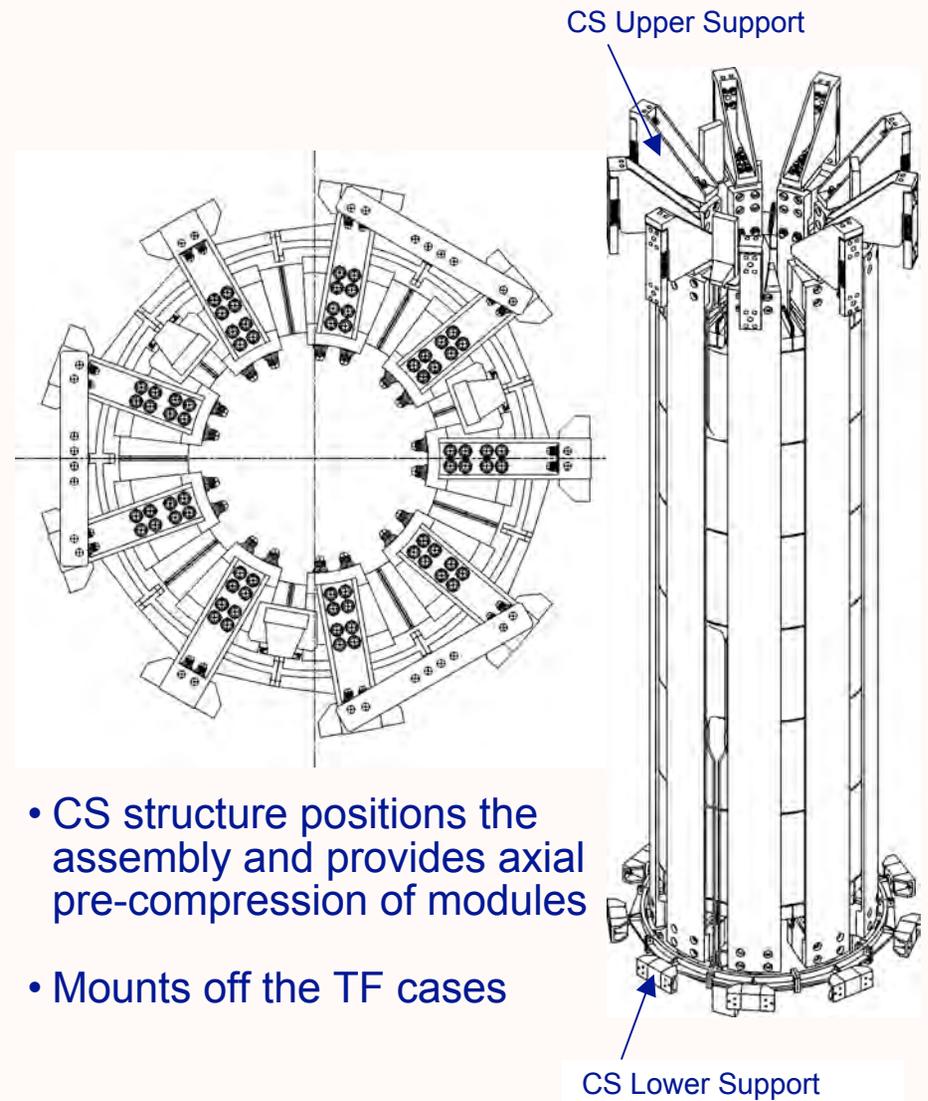
- 9 “long” double-pancake lengths (about 765 m each) + 1 dummy length
- Nearly 7 km of “active” CICC
- Nearly 38 t of  $\text{Nb}_3\text{Sn}$  wire

# Central Solenoid



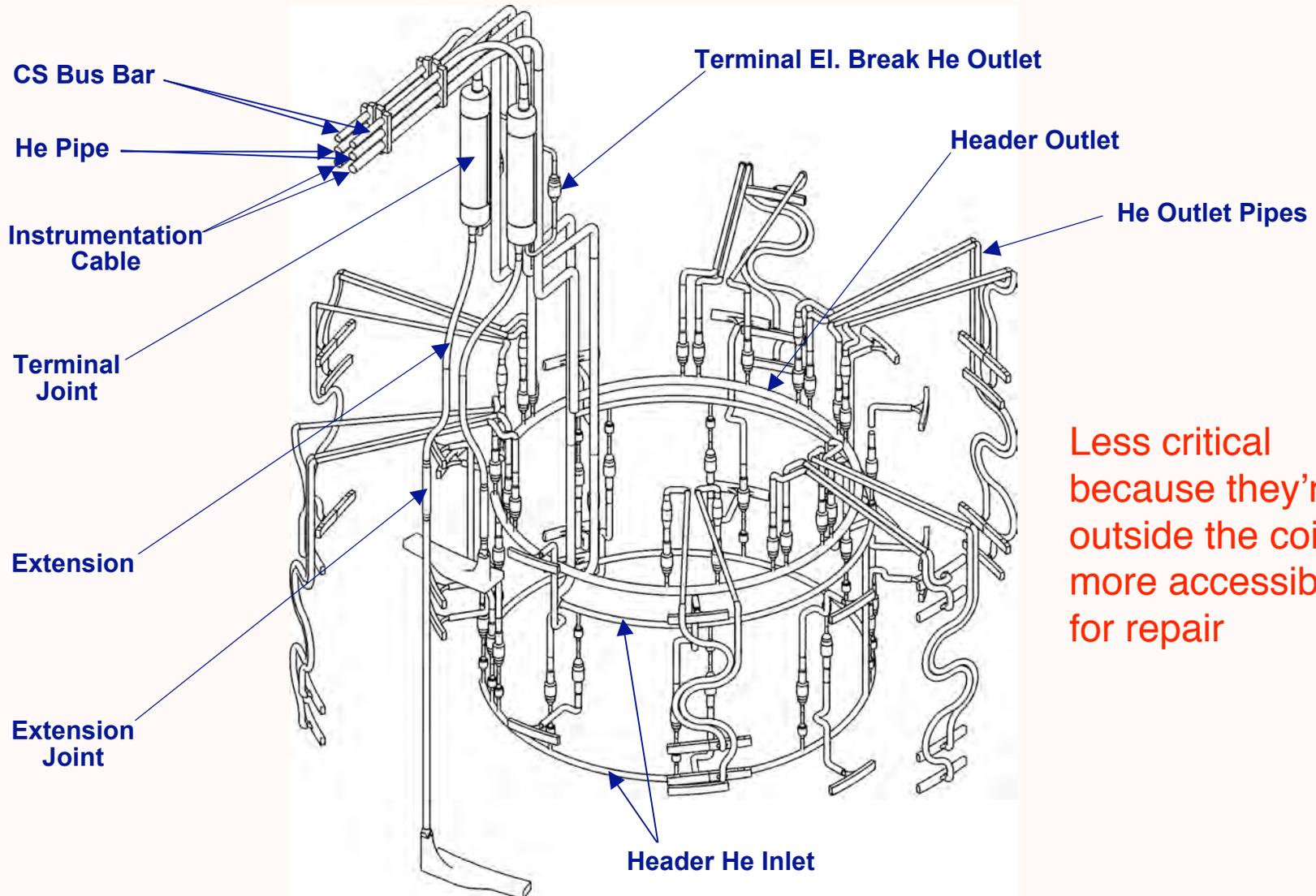
## CS Coil Modules

- 6 modules + 1 spare
- All identical
- In-line conductor butt joints between sections



- CS structure positions the assembly and provides axial pre-compression of modules
- Mounts off the TF cases

# Scope – CS External Piping, Extensions & Joints



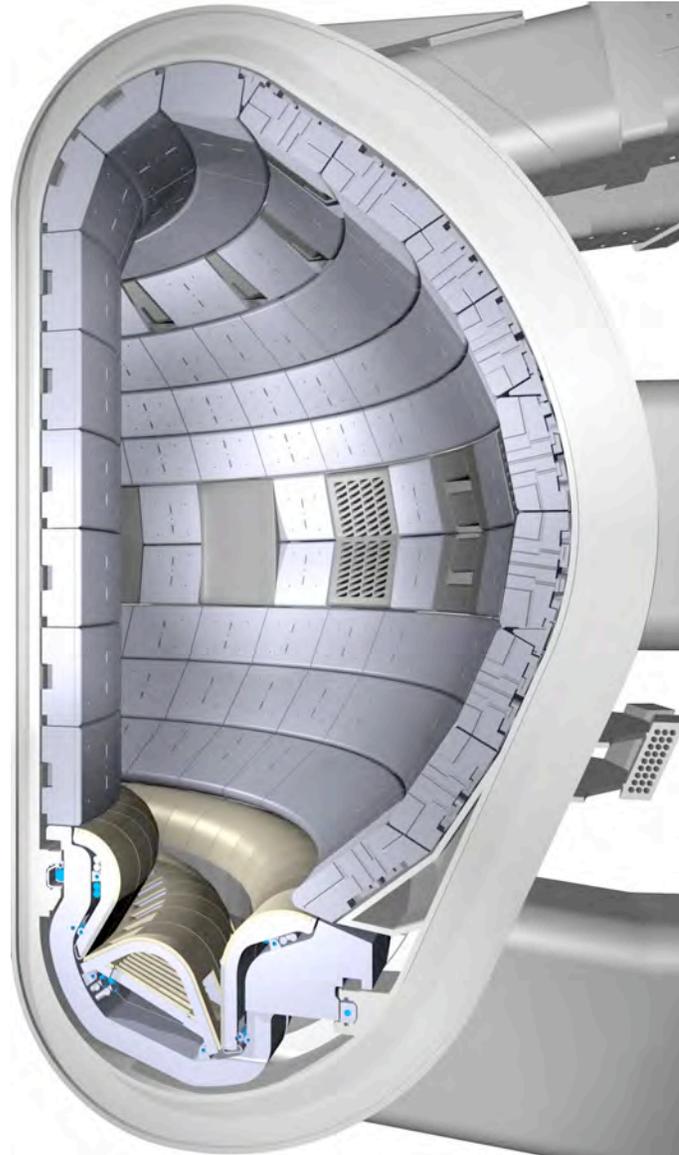
Less critical because they're outside the coil, more accessible for repair

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# Power-handling

# In-vessel structures

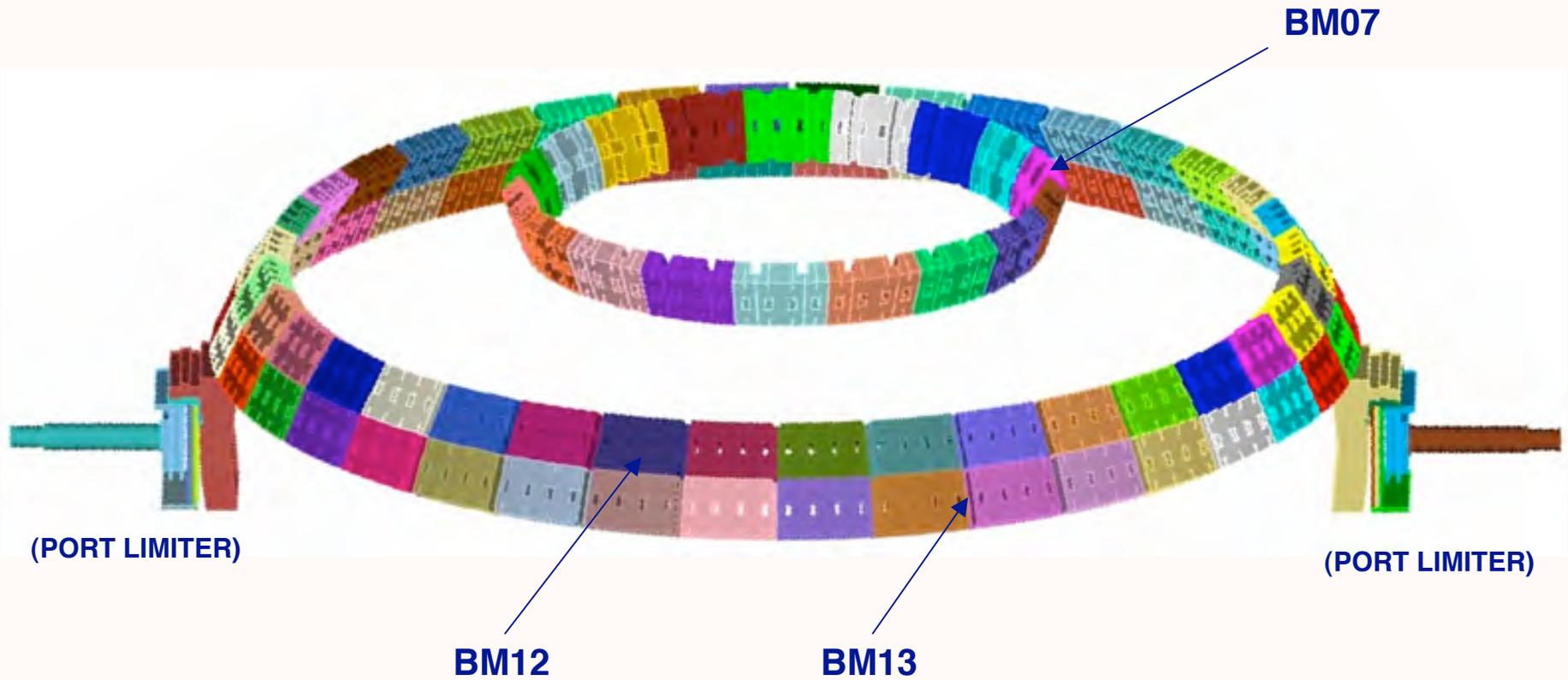
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# Scope – Blanket Module



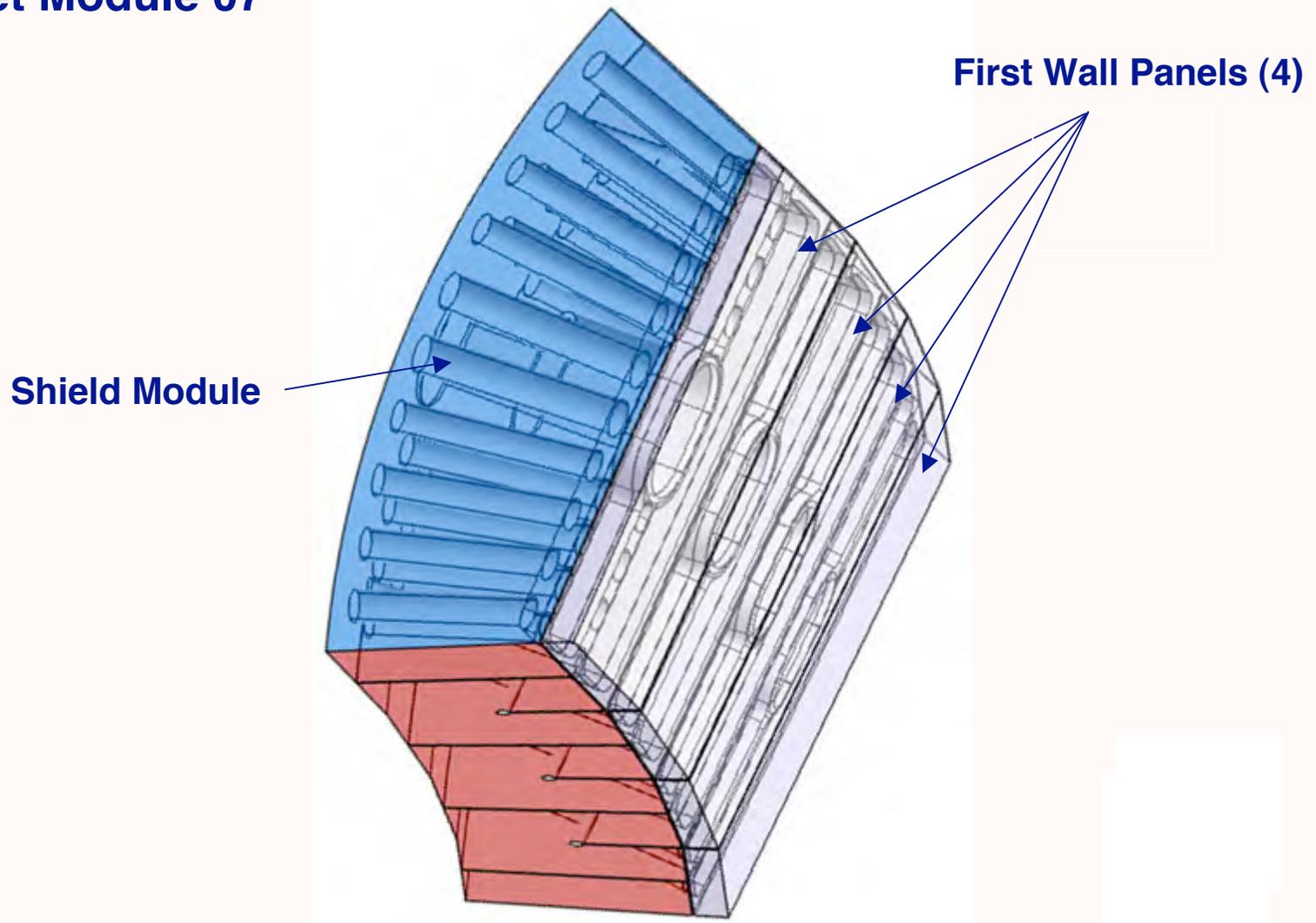
## Module Allocation



# Scope – Blanket Module



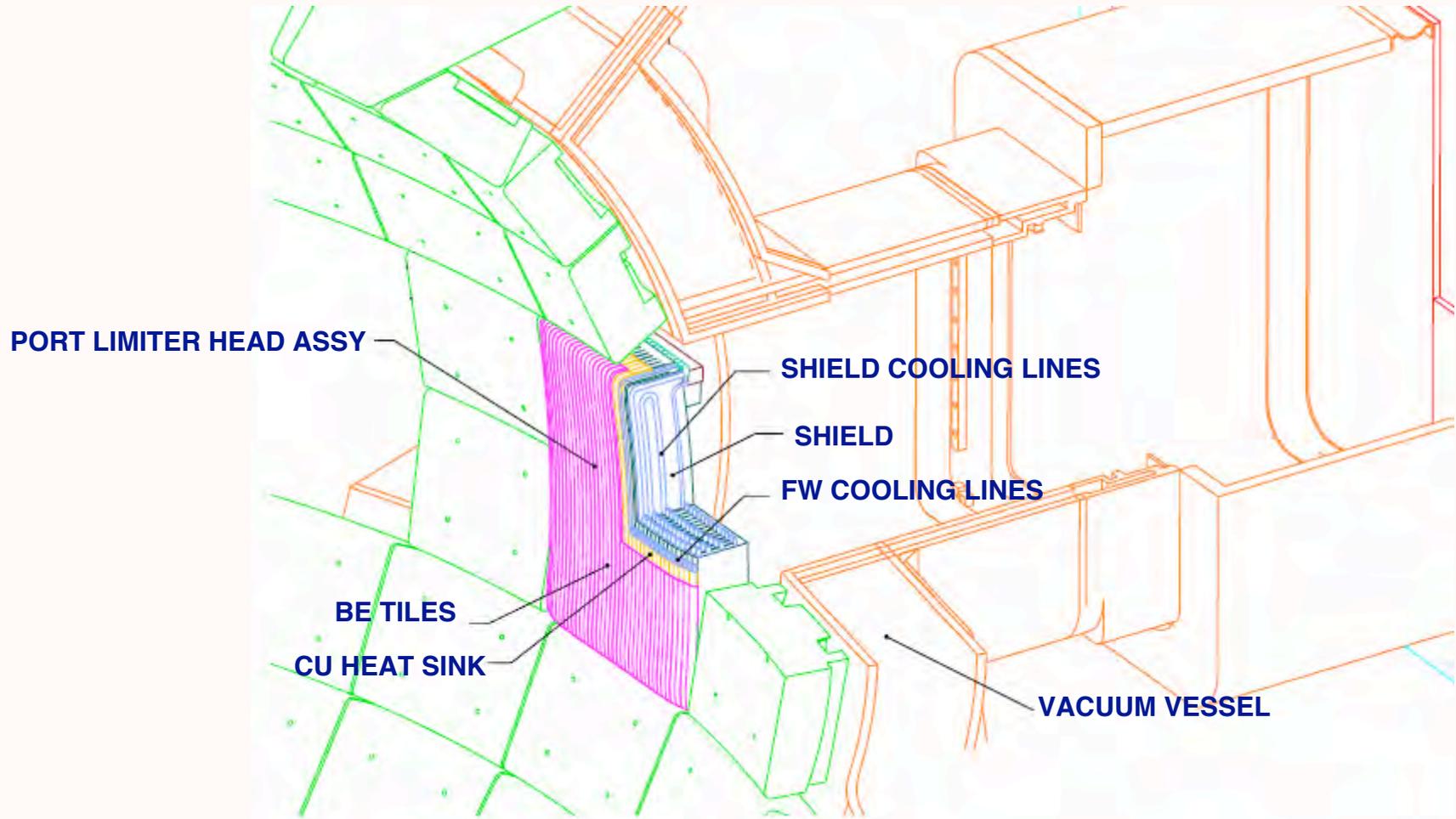
## Blanket Module 07



# Port Limiter



## Plasma Facing Head Orientation



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# Plasma control, heating, current drive

# ICH system

The ITER ICH system will deliver 20 MW of power to the plasma for ion heating and for central current drive

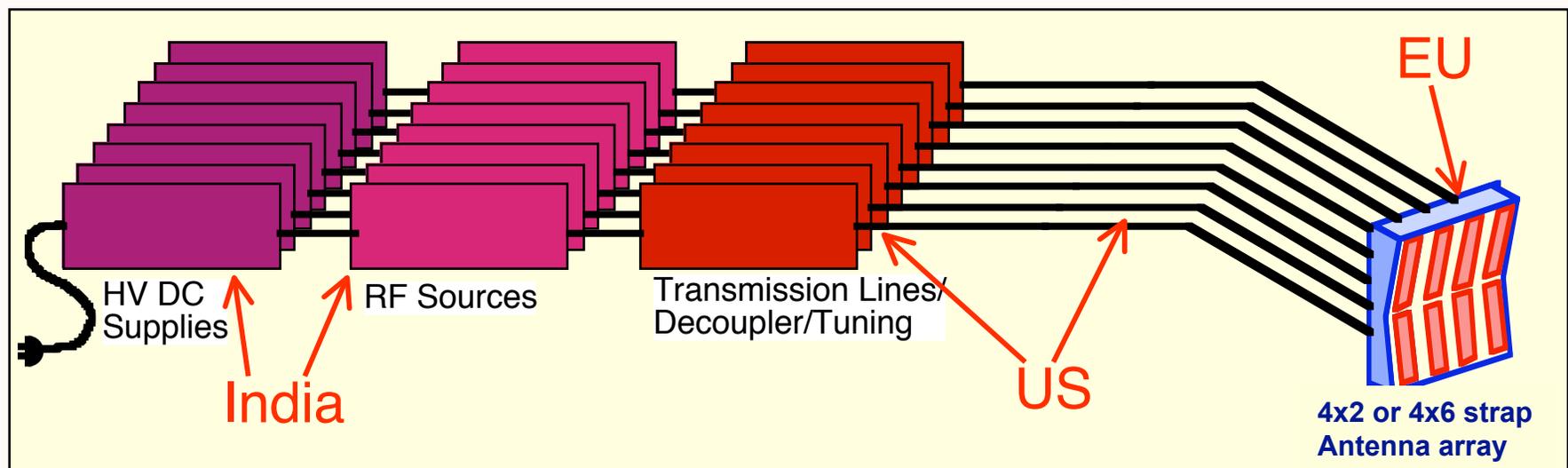
It is:

- One antenna, 8 or 24 current straps
- Eight rf sources, each feeding one set of straps in the antenna
- 40-55 MHz frequency range
- Adjustable phasing between straps for heating or current drive

It can be used for:

- Tritium ion heating during DT ops
- Minority ion heating with initial H/D ops
- Central current drive for AT ops
- Minority ion current drive at sawtooth inversion radius

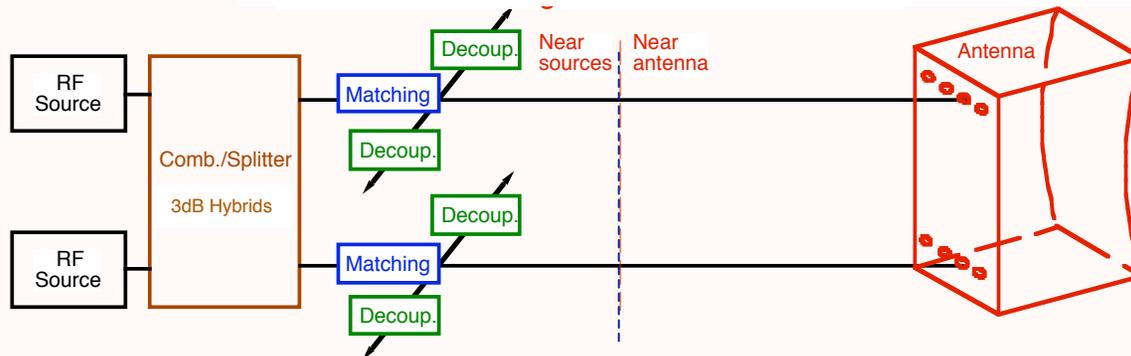
ITER ion cyclotron system block diagram



# Issue: Matching system depends on antenna, impacts component, quantity details and risk



2001 Internal-match baseline design – load-tolerant antenna

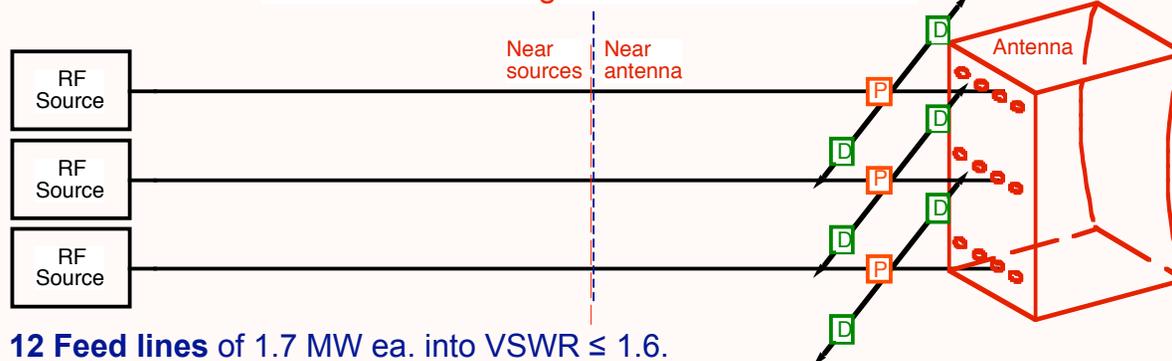


**8 Feed lines** of 2.5 MW ea. into  $VSWR \leq 2.0$ .

Antenna is load-tolerant with internal matching components (sliding stub-tuned triax).

**Risk:** 5MW section connects combiner and splitter. 2.5 MW center conductor cooling required. Antenna performance and reliability is now deemed inadequate

Internal-match design – load-tolerant antenna



**12 Feed lines** of 1.7 MW ea. into  $VSWR \leq 1.6$ .

Antenna is load-tolerant with internal matching components. Pre-match keeps  $VSWR \leq 1.6$ .

Hybrid not used since it is incompatible with phasing requirements.

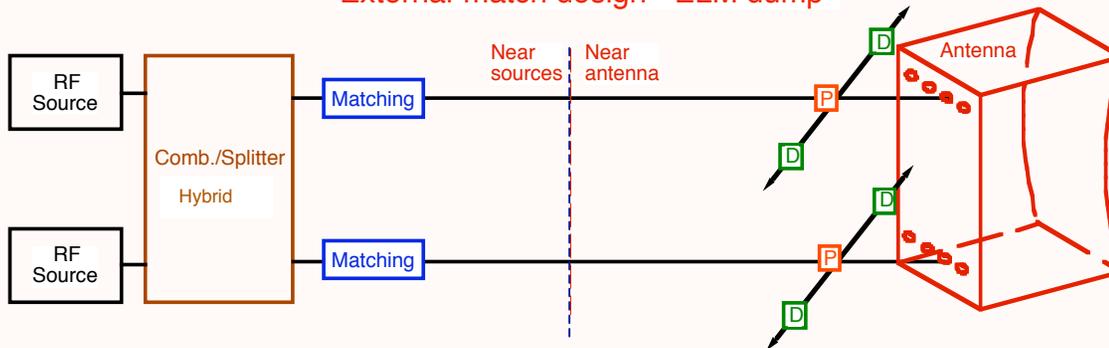
Internal tuning similar to JET-EP and Tore Supra load tolerant antennas.

**Risk:** 12 feed lines increase transmission line & matching cost. Smaller lines still need inner cooling (1.7 MW).

# Issue: Matching system depends on antenna, impacts component, quantity details and risk



External-match design -ELM dump



**8 Feed lines** of 2.5 MW ea. into  $VSWR \leq 1.3$ .

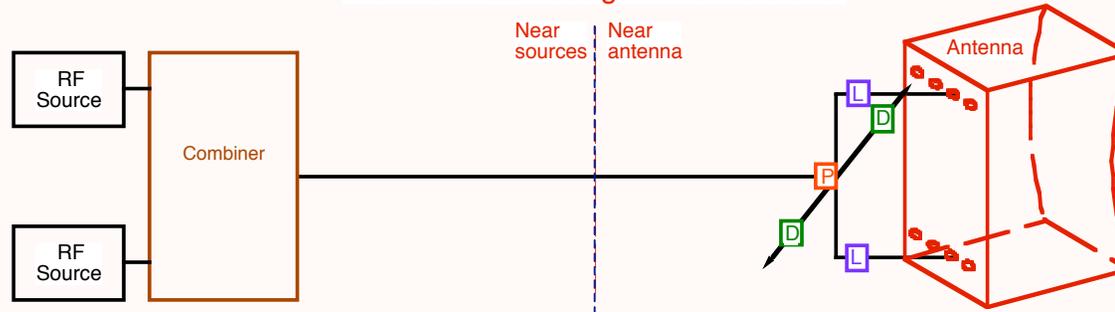
Antenna not load-tolerant, simple design with no moving parts. Pre-match keeps  $VSWR \leq 1.6$ .

Reflected power (plasma load variations) absorbed in dump resistors, not available for plasma heating.

Rearrangement of baseline components with minor changes

**Risk:** Some loss of power/performance during ELMs. Lowest risk option for transmission line and matching

External-match design – load-tolerant



**4 Feed lines** of 5 MW ea. into  $VSWR \leq 1.6$ .

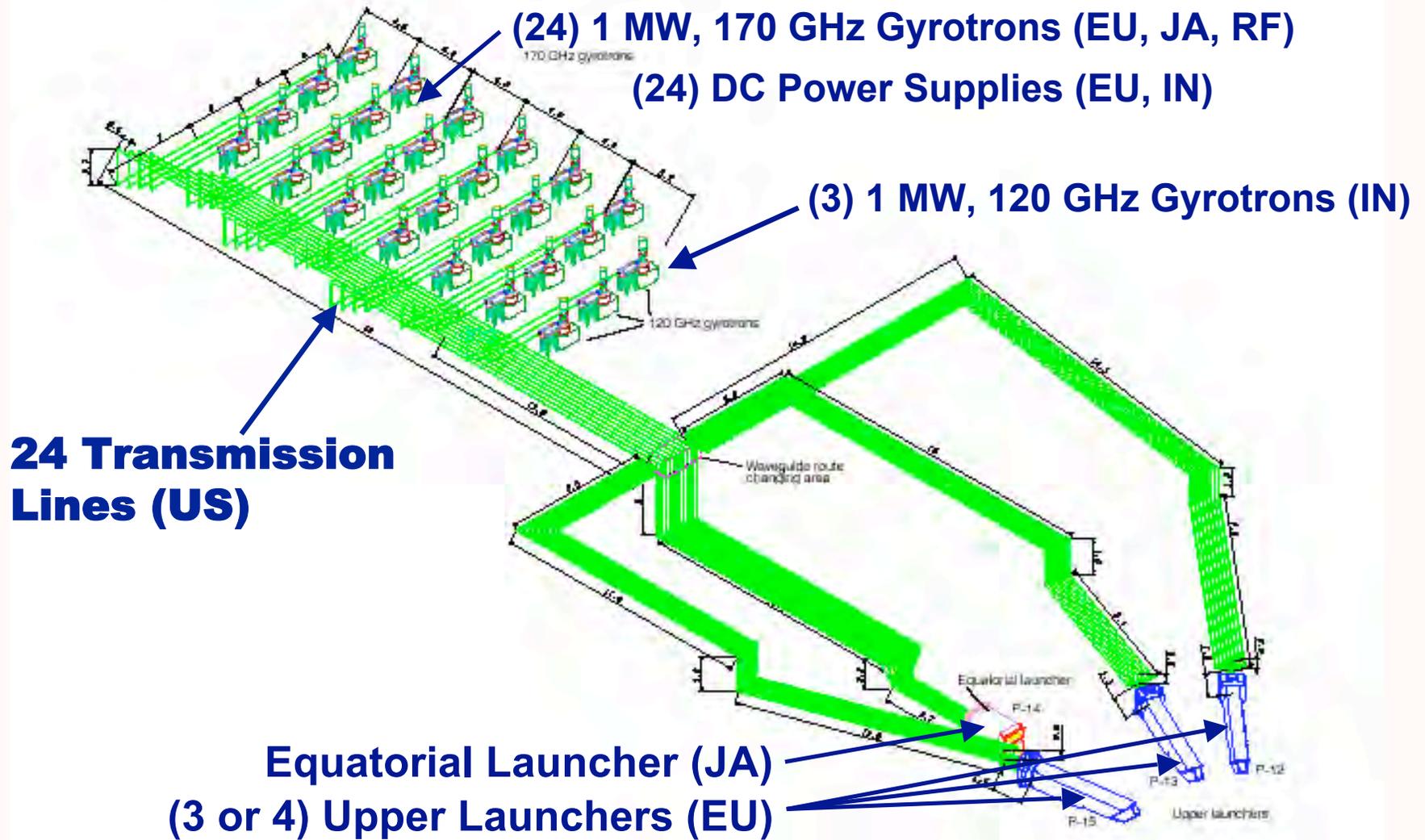
Antenna load tolerant with no moving parts, but requires significant external components near antenna.

Pre-match keeps  $VSWR \leq 1.6$ . May need additional tuning components near antenna.

Tuning architecture to be tested on JET A2 antenna

**Risk:** Extended length of high power line (5 MW). More matching components required.

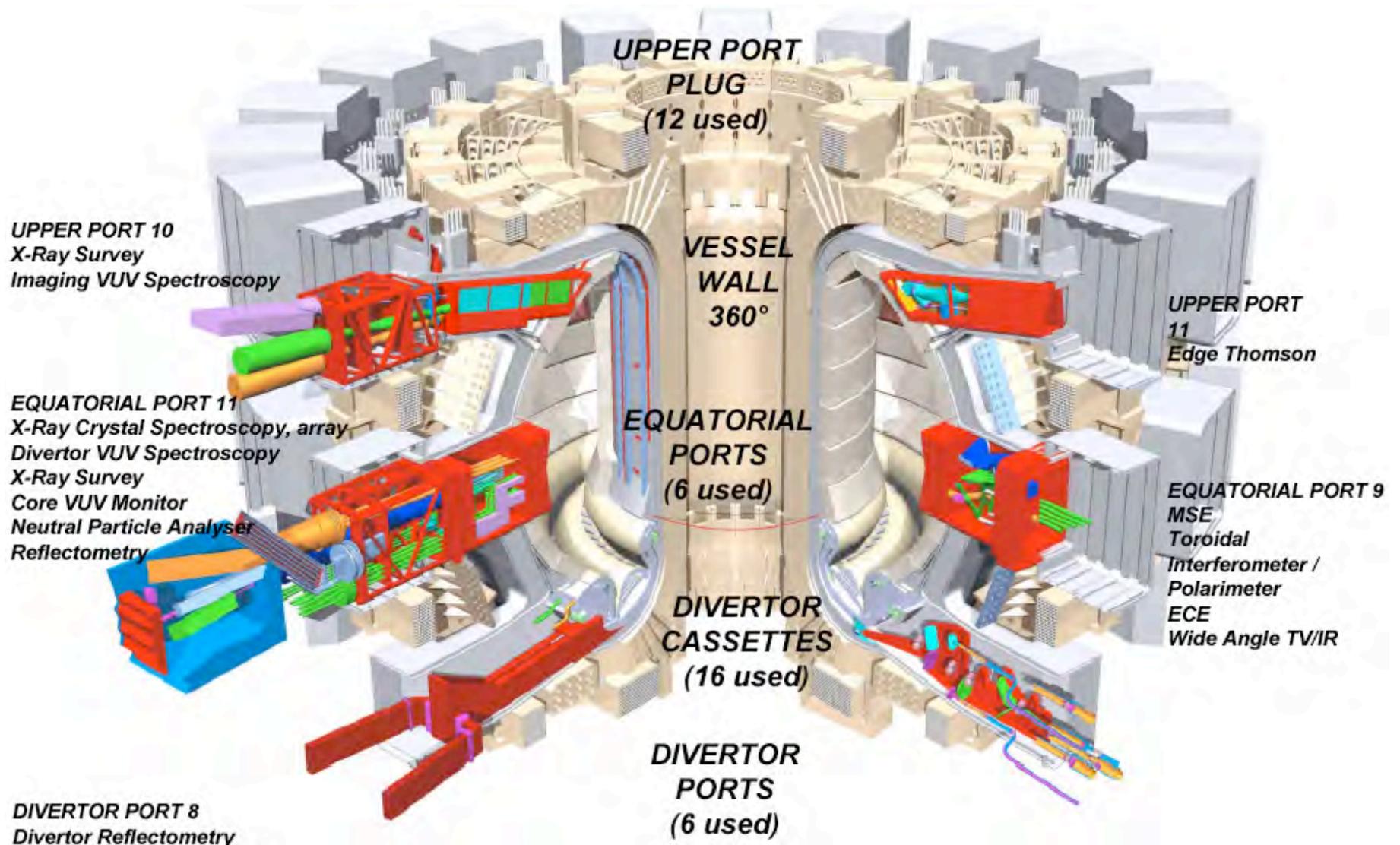
# ECH&CD Transmission Lines



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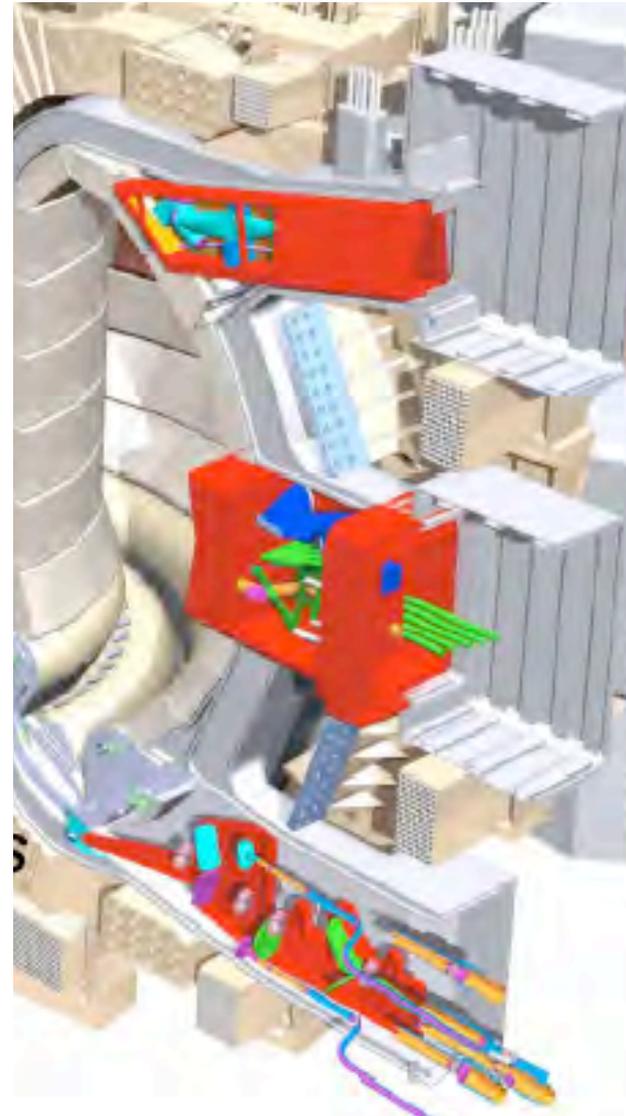
# Diagnostic instrumentation

# Instrumentation is key to science on ITER



# A Significant (16%) Diagnostic Role in ITER Supports a Strong Research Presence

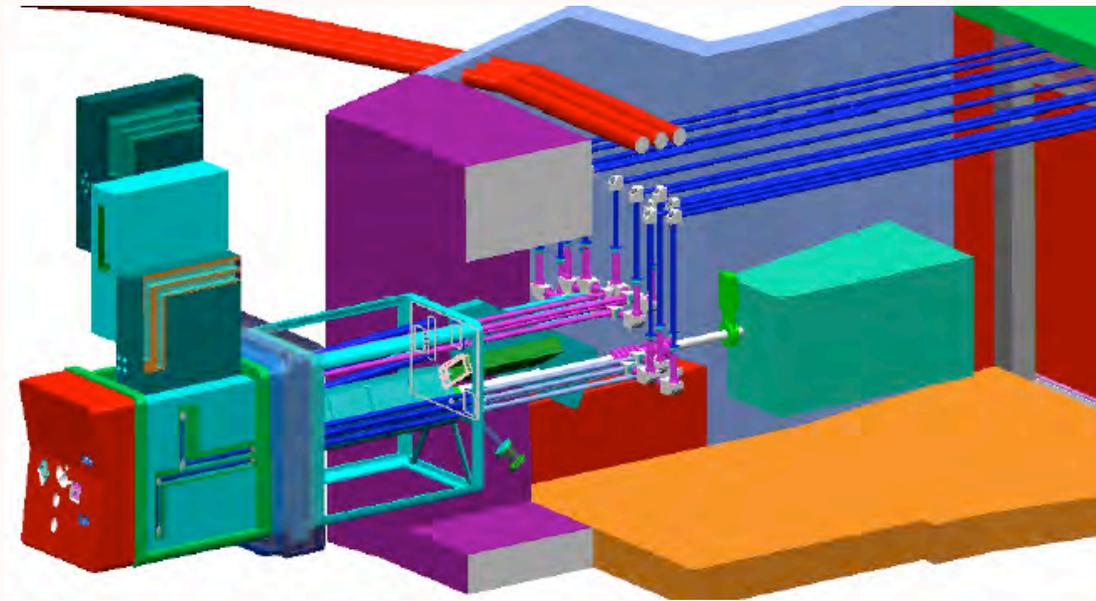
- **Seven Diagnostic Systems**
  - Motional Stark effect polarimeter
  - 6 visible/IR cameras in upper ports
  - Low-field-side reflectometer
  - Electron cyclotron emission
  - Tangential interferometer/polarimeter
  - Divertor interferometer
  - Residual gas analyzer
- **Five Diagnostic Port Plug Structures**
  - 2 upper plugs
  - 2 equatorial plugs
  - 1 lower plug structure



# Instrumentation Example Reflectometer



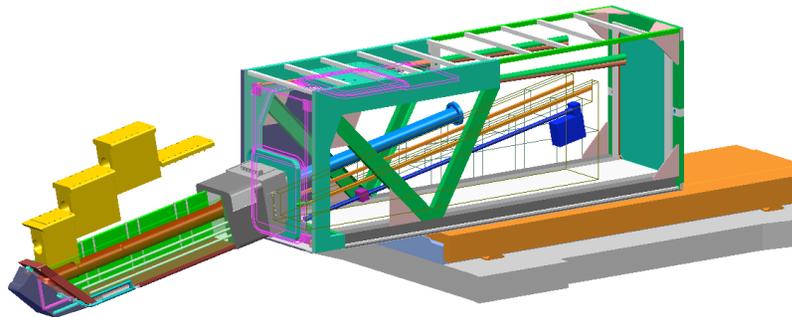
- Designed to measure density profiles with high time resolution and density fluctuations.



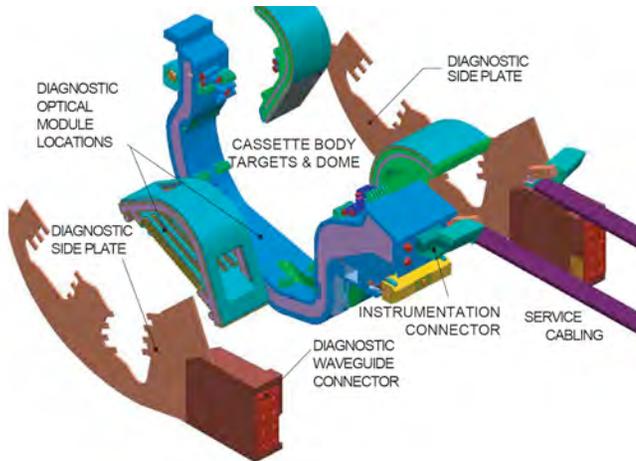
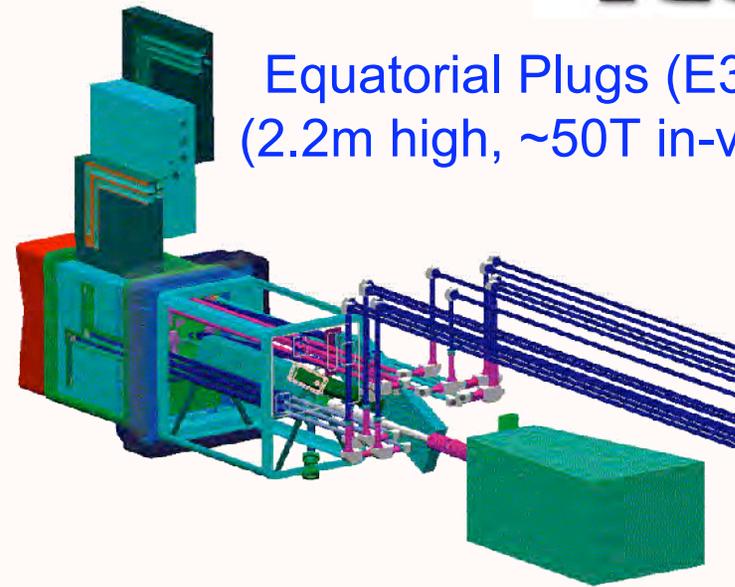
- Instrumentation packages consist of
  - Front-end components embedded in port plug
    - mirrors, waveguides/horns, shutters, calibration sources
  - Ex-cryostat components some of which are far removed
    - fiber optics, transmission lines, sources, detectors

# Scope - US Port Plugs

Upper Plugs (U5, U17)  
(4.5m long, ~25T in-vessel)



Equatorial Plugs (E3, E9)  
(2.2m high, ~50T in-vessel)

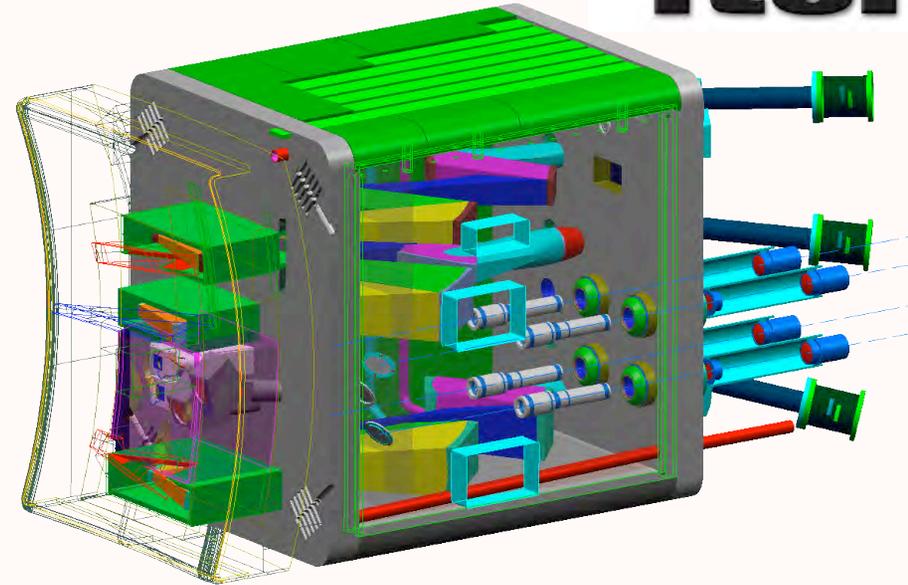


Divertor Side Panels  
and Back Boxes (L8)

Plugs provide

- Vacuum seal, radiation shielding
- Cooling water and support for blanket shield modules
- Support and access for diagnostics

# Scope - Integration of Diagnostics into Plugs



- Diagnostics from other parties need to be integrated into plug E3
  - 2 visible/IR camera views (EU)
  - 2 edge CXRS views (RF)
  - 2  $H_{\alpha}$  arrays (RF)
  - X-ray crystal views (IN?)
- The division of integration design responsibility between the IT and the parties has not been fully negotiated.

# Preparations Leading to Major Diagnostic Design Effort for US Diagnostic Systems

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- **Over the next 6-9 months, USIPO will be assisting the ITER Organization to prepare procurement documents to more clearly define the US diagnostic scope.**
  - First award for major design of a US system presently scheduled for early FY08.
- **A number of “assessment studies” were recently launched to re-engage US diagnostic experts.**
  - Resulting performance assessments and revised cost estimates will help guide procurement arrangements.
- **Core engineering team will assist US diagnostic designers to integrate instrument front-ends into the plugs.**

# Ongoing Studies Assess Reference Designs for US Systems

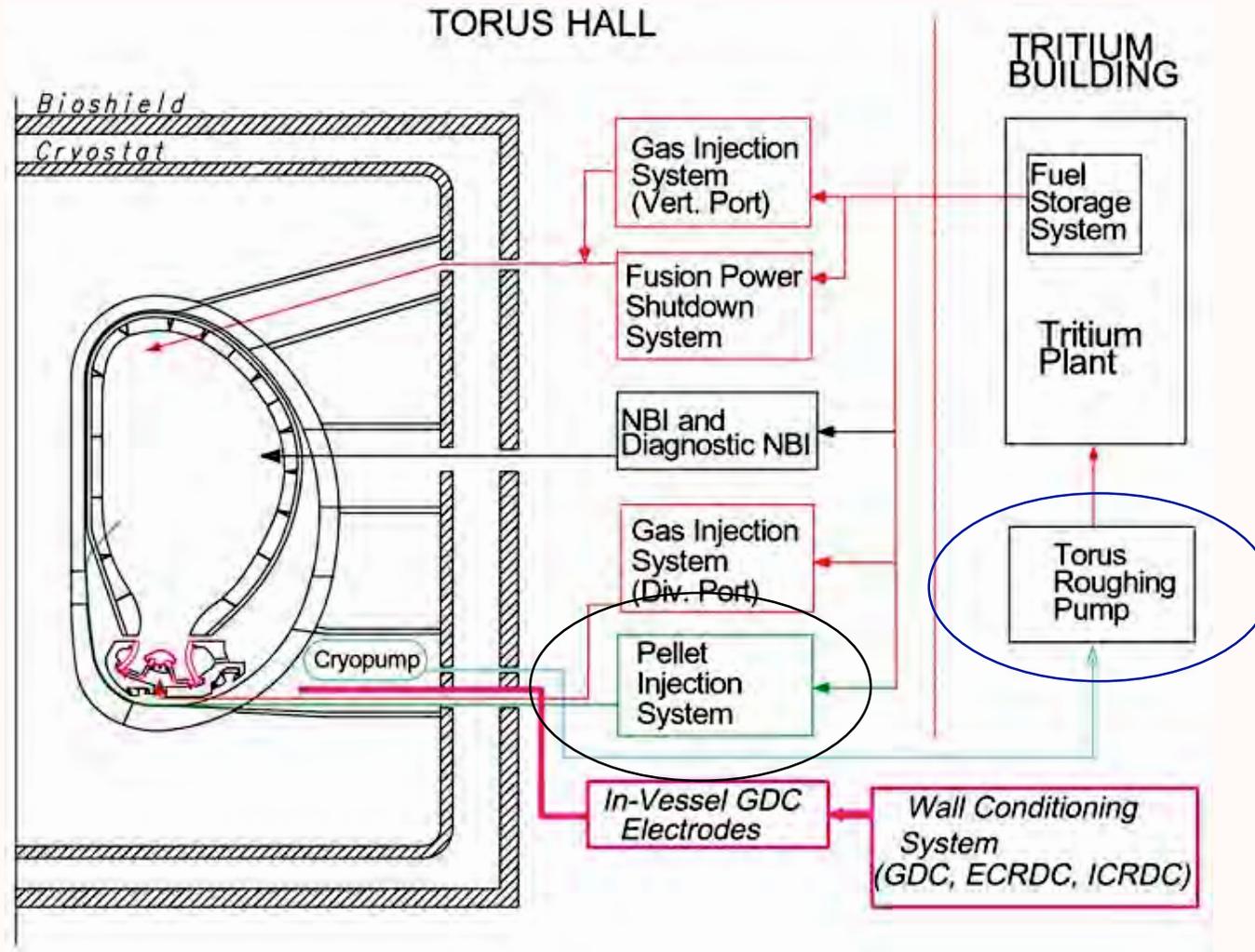
Diagnostic Package	Task Summary	Institution(s)
Upper Visible/IR Camera	assess optical design, central tube concept	LLNL
LFS Reflectometer	determine optimum frequency bands and polarizations	UCLA
MSE	assess usefulness of  B  determination	NOVA
MSE	performance simulation of conventional polarimetry approach	PPPL
MSE	optimization of optical design	LLNL
ECE	investigate non-thermal issues, use of oblique view	PPPL
ECE	review reference design	U. Texas-U. Md-MIT
Divertor Interferometer	develop conceptual design	GA-UCLA
RGA	develop conceptual design	ORNL
Tang. Interfer./Polarimeter	optimize reference design	UCLA-GA
First Mirror R&D	model erosion/deposition on 1st mirrors	ANL
Neutronics Analysis	benchmark neutronics models for plug integration using ATILLA	UCLA-PPPL

- **Focus on identification of ‘front-end’ configurations**
- **Upcoming meetings provide opportunity for broader input**
  - USBPO Workshop 6 - 8 February 2007 at General Atomics.
  - ITPA Diagnostics TG Meeting 26 - 30 March 2007 at PPPL.

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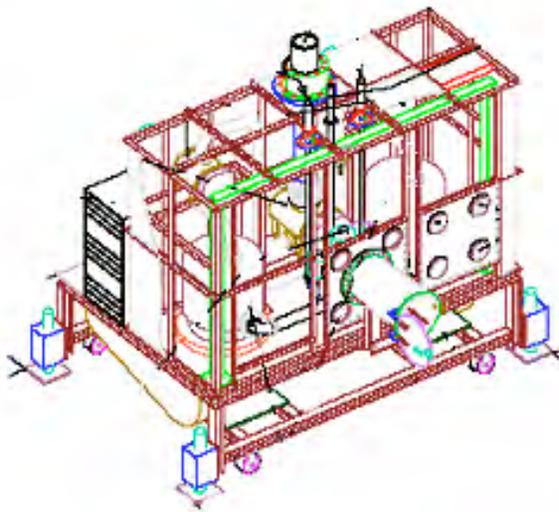
# Fuelling and exhaust processing

# ITER Pumping and Fueling Systems

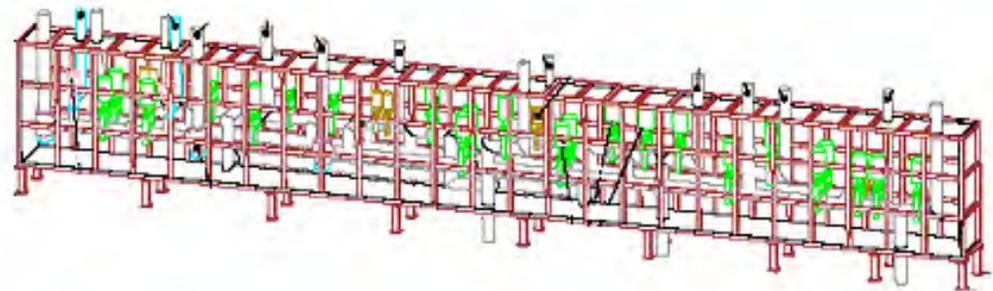


# ITER Roughing Pump Sets

- **Roughing pump sets - 4 identical pump assemblies**  
Piston pumps, Blowers, mounted in glove box assembly with associated valves, instrumentation and controls



**Roughing Pump Set  
Assembly**



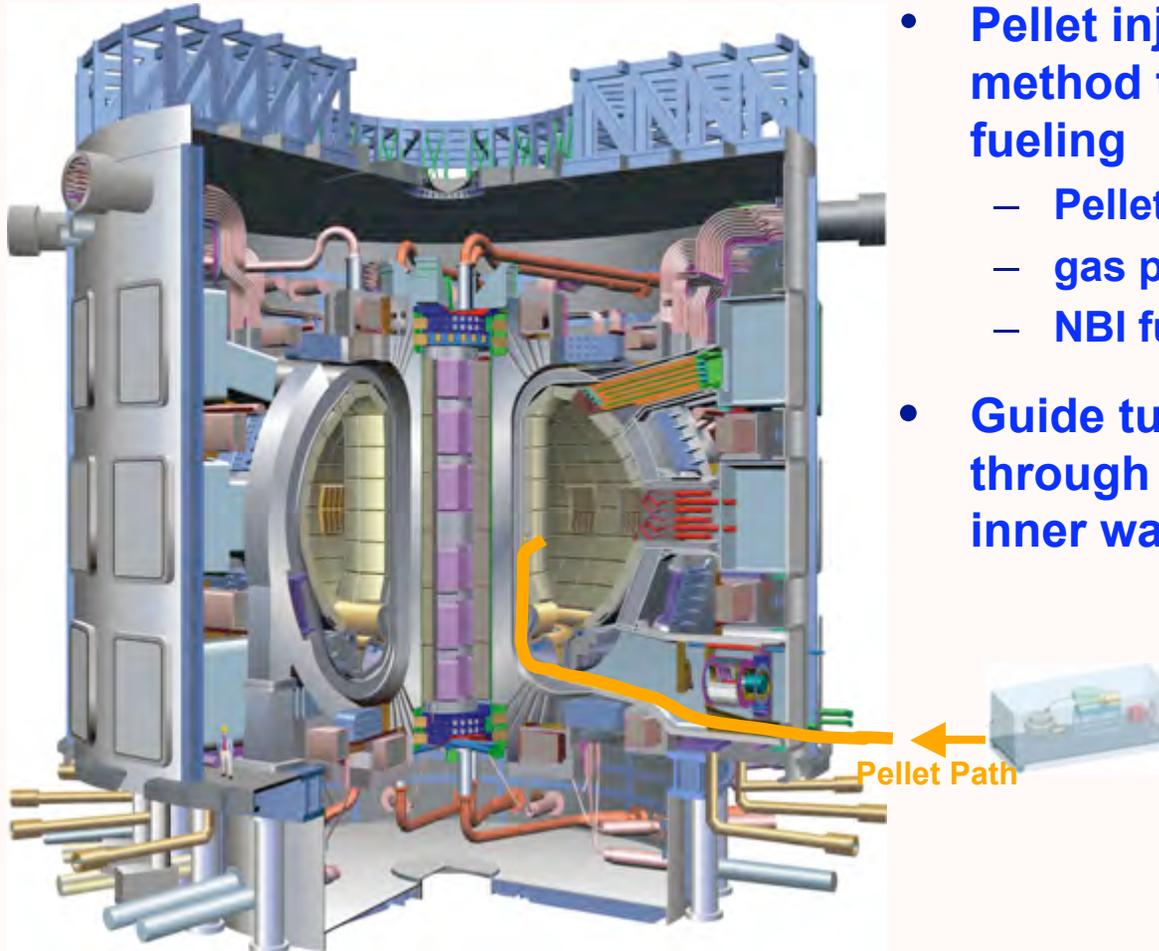
**Change Over Valve Box Assembly**

# ITER Vacuum Standard Components



- **Standard components consists of**
  - **ICRF vacuum system - 64 getter pumps and 32 valves**
  - **ECH vacuum system - 130 sputter ion pumps, 10 TMPs, 10 dry pumps & 220 valves**
  - **Guard and service vacuum system - 86 cryo pumps, 2 dry pumps and 1738 valves**

# Pellet Injector

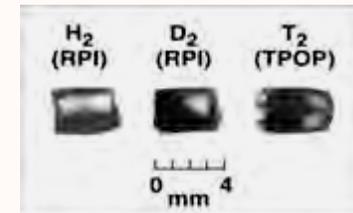


- Pellet injection the only known method to achieve efficient core  $T_2$  fueling
  - Pellets ~90% efficient
  - gas puffing < 1% efficient
  - NBI fueling negligible
- Guide tubes bring the pellets through the divertor ports to the inner wall.

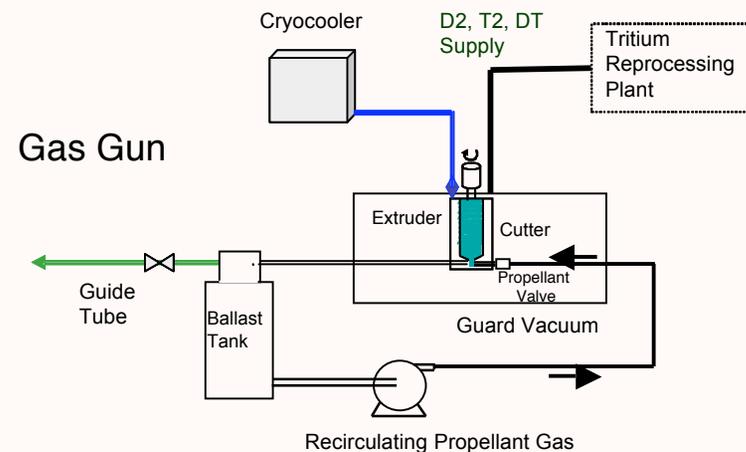
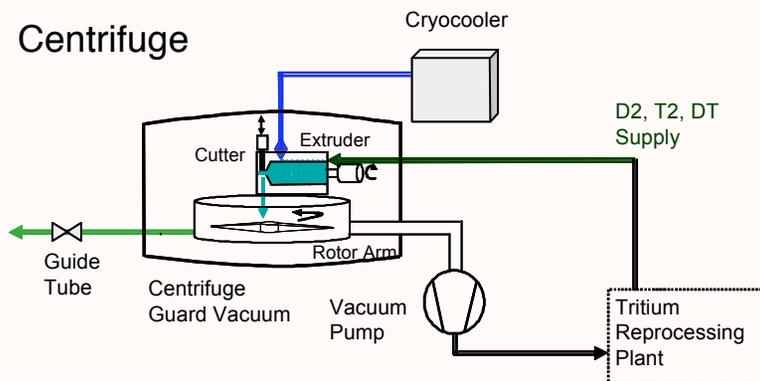
# Pellet Injector Design

## Two concepts are available for ITER pellet acceleration

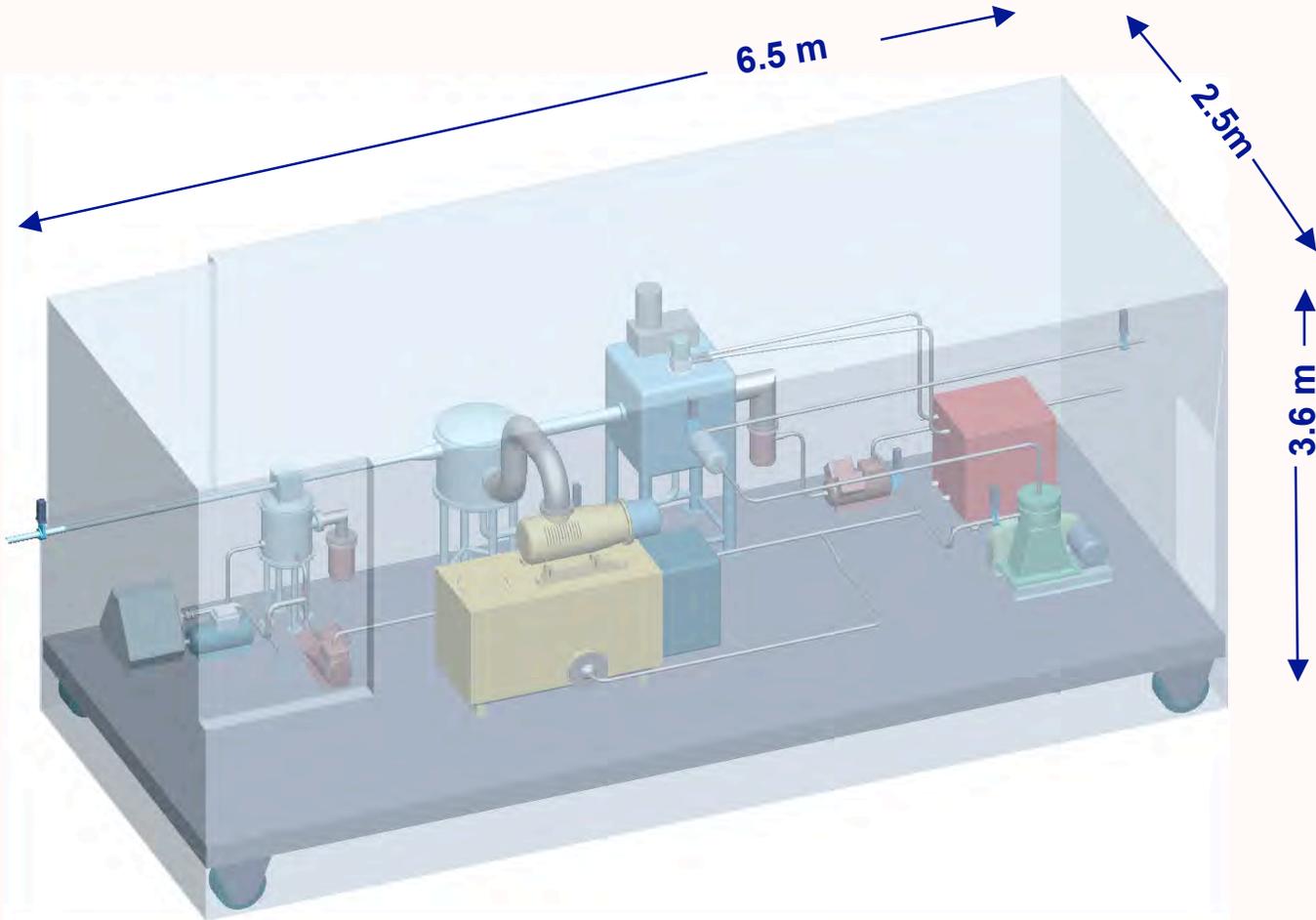
- ITER baseline design assumes a centrifuge accelerator
  - Low (~10%) recirculating gas from accelerator
  - Existing designs do not meet the needed reliability
  - Significant development needed to meet the ITER reliability requirements
- Gas gun technology may be a better choice
  - High reliability has been demonstrated for slow pellets
  - Propellant gas valve can be optimized for low gas usage
  - A recirculating propellant gas system can be employed to minimize impact on the tritium plant



Hydrogen, Deuterium and Tritium Pellets

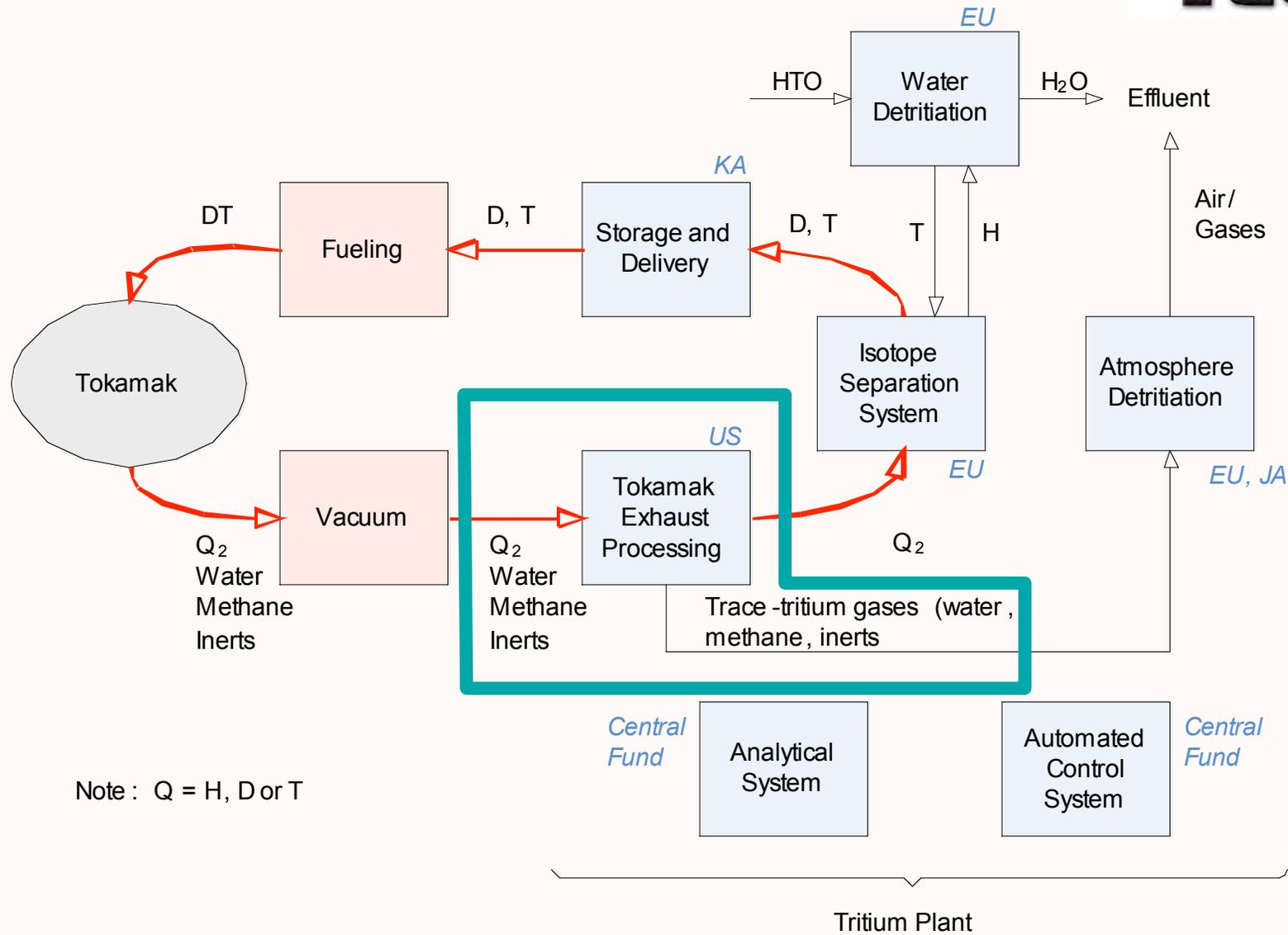


# ITER Gas Gun Pellet Injection System Conceptual Design

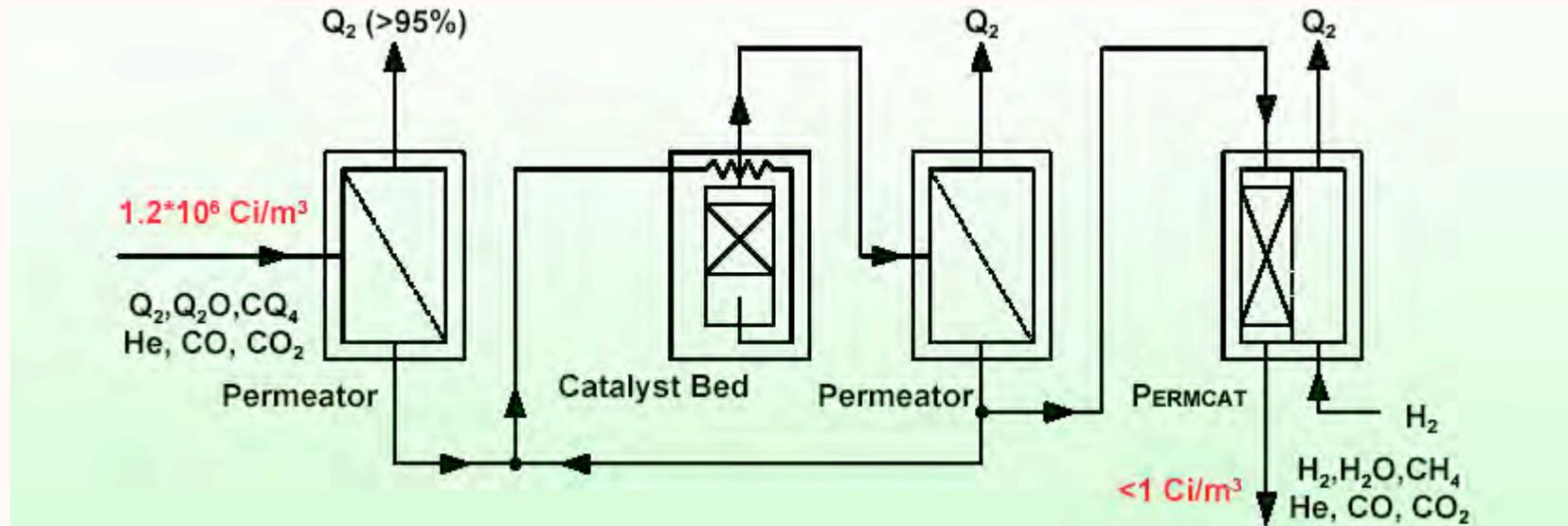


Cask layout from P. Fogarty, ORNL

# Tritium Processing System

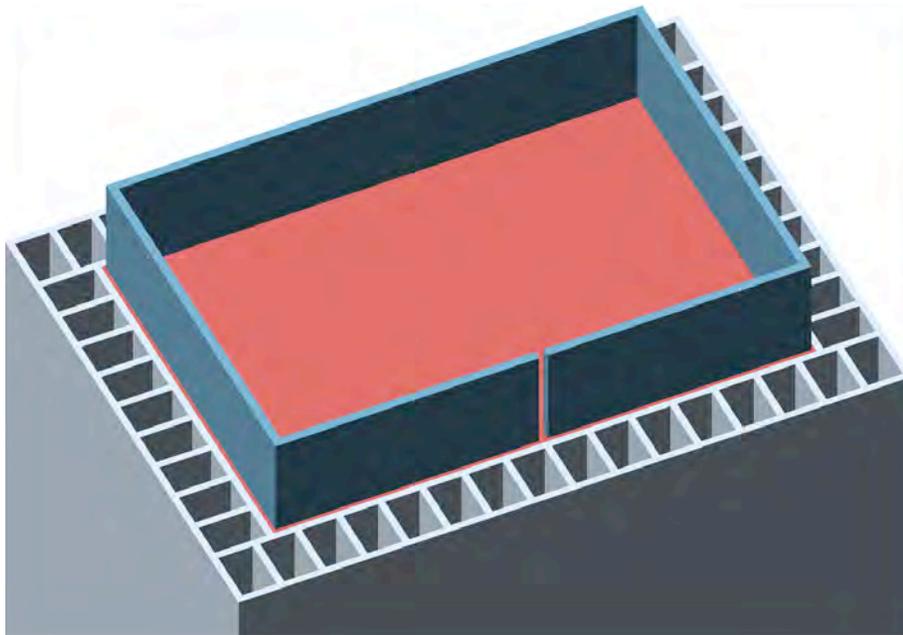


# Steps of the Tokamak Exhaust Processing

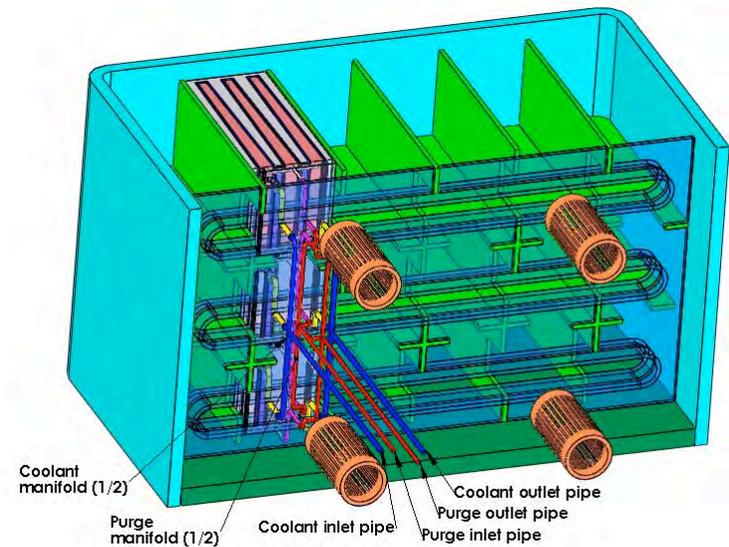


- Battery of front-end permeators (first process step)
- Cracking or conversion reactions (impurity processing, second process step)
- Counter current isotopic swamping (final clean-up, third process step)

# Tritium-breeding: Test Blanket Modules (Outside the ITER Construction Project and US ITER Project)



*Dual Coolant Lead- Lithium TBM*

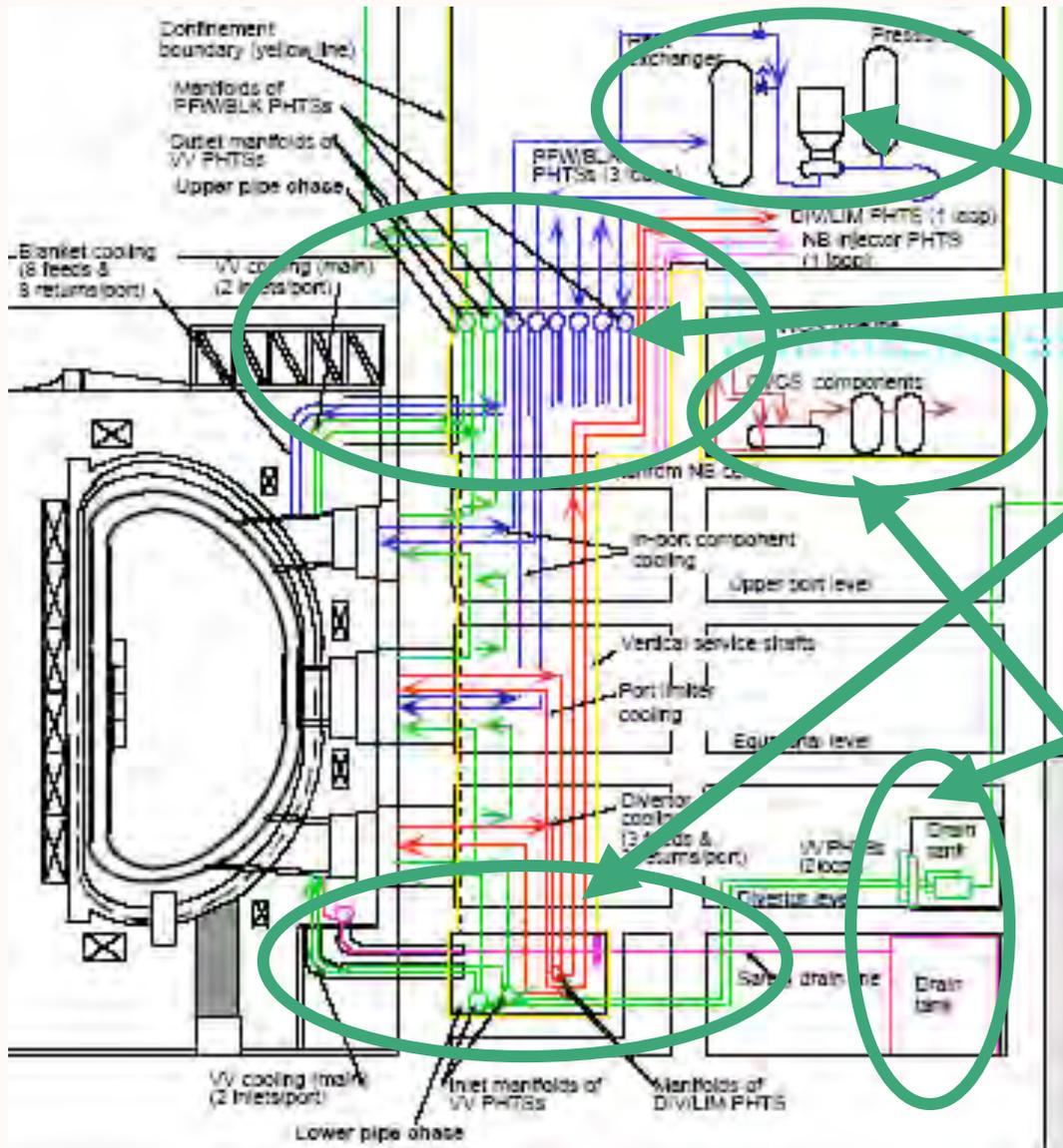


*Schematic view of solid breeder thermomechanics unit cell test articles housed inside helium-cooled pebble bed box*

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# Cooling Water

# Seven sub-systems distributed throughout Tokamak building



## Primary Heat Transfer:

- Vault
- Upper Pipe Chase
- Roof
- Lower Pipe Chase

## Support Systems:

- Draining and Refilling (WBS1.2.1.6)
- Chemical and Volume Control System (WBS1.2.1.5)
- Drying (WBS1.2.1.7)

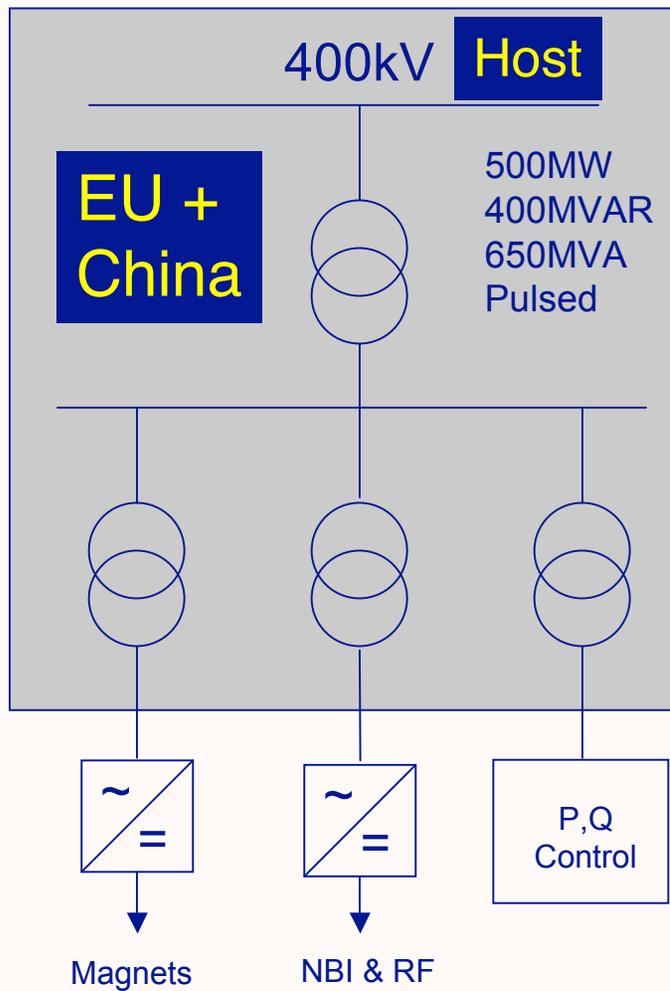
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# Steady-state Electric Power

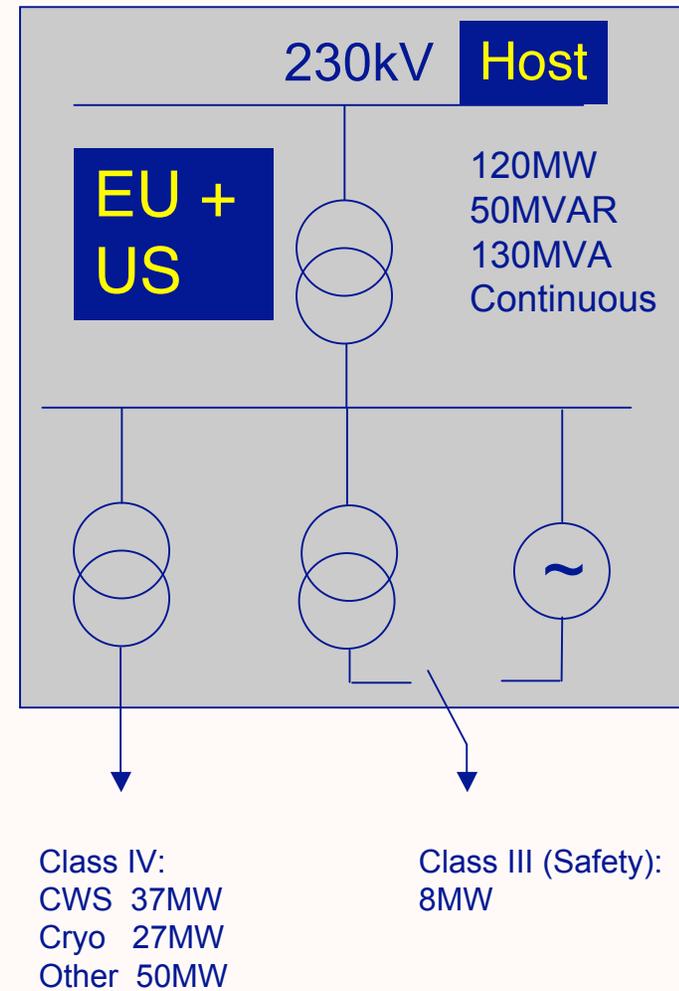
# ITER AC Power Systems: Generic Site



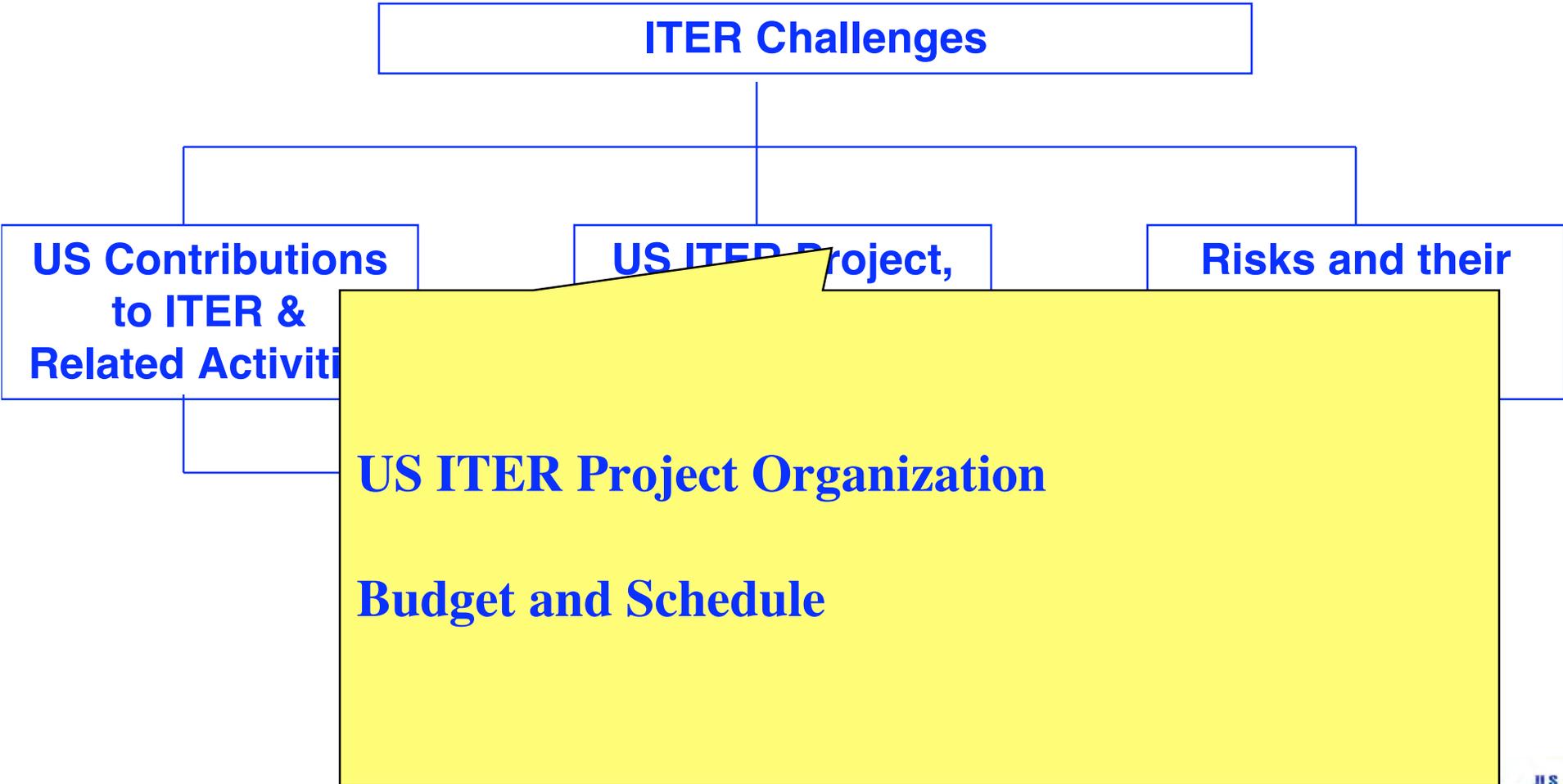
### Pulsed Power Electric Network



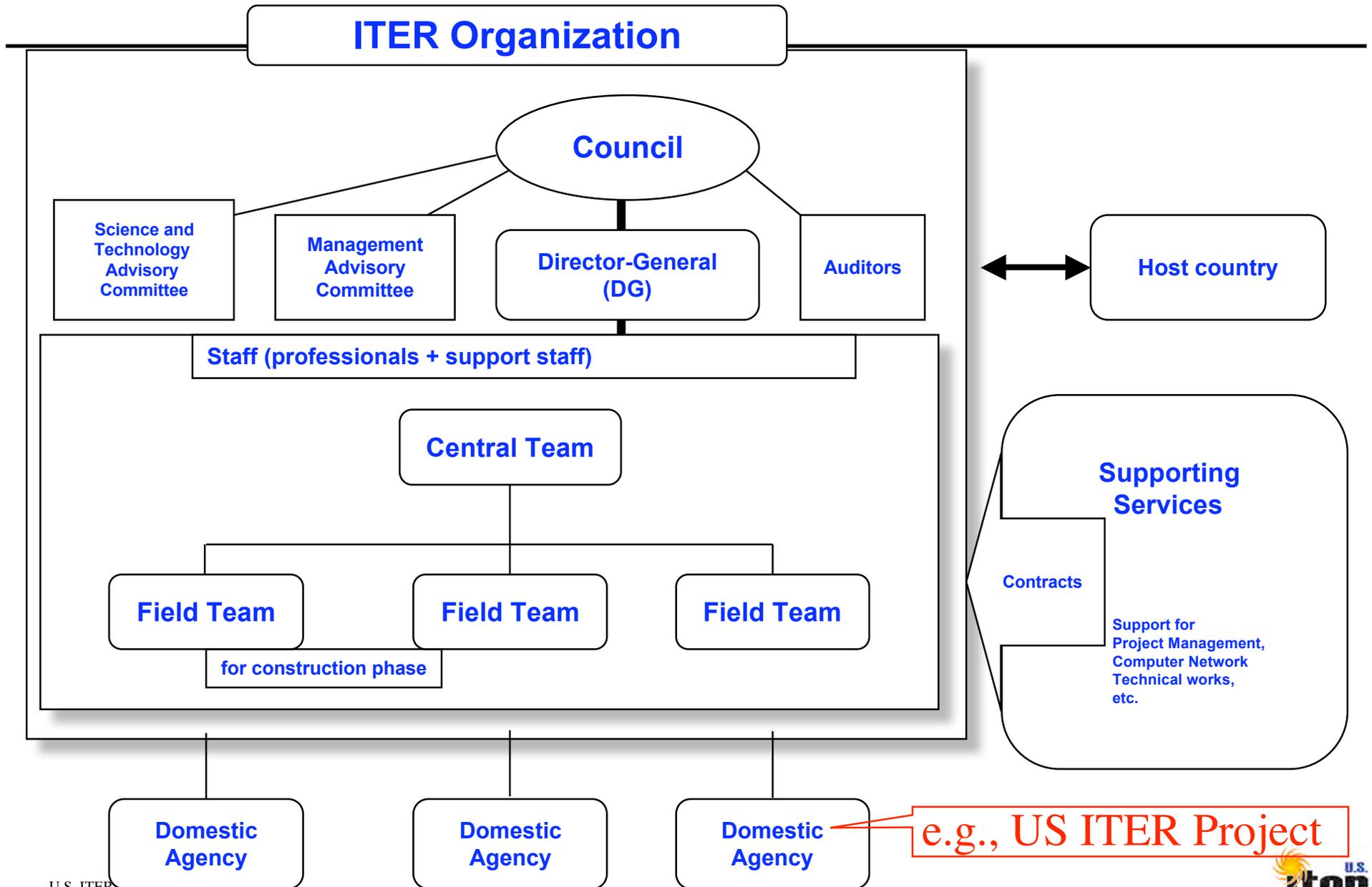
### Steady State Electric Power Network



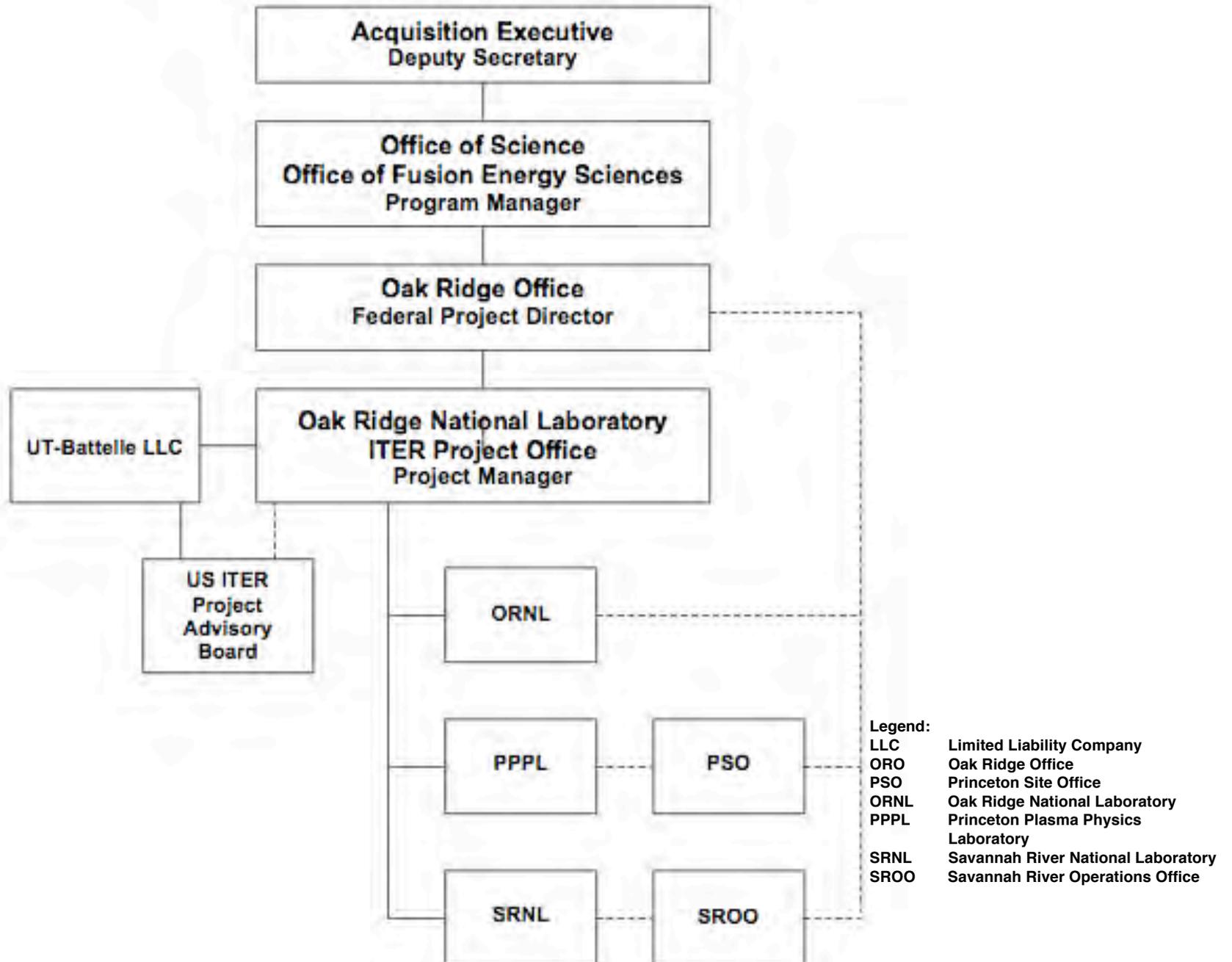
# Structure of the Talk...



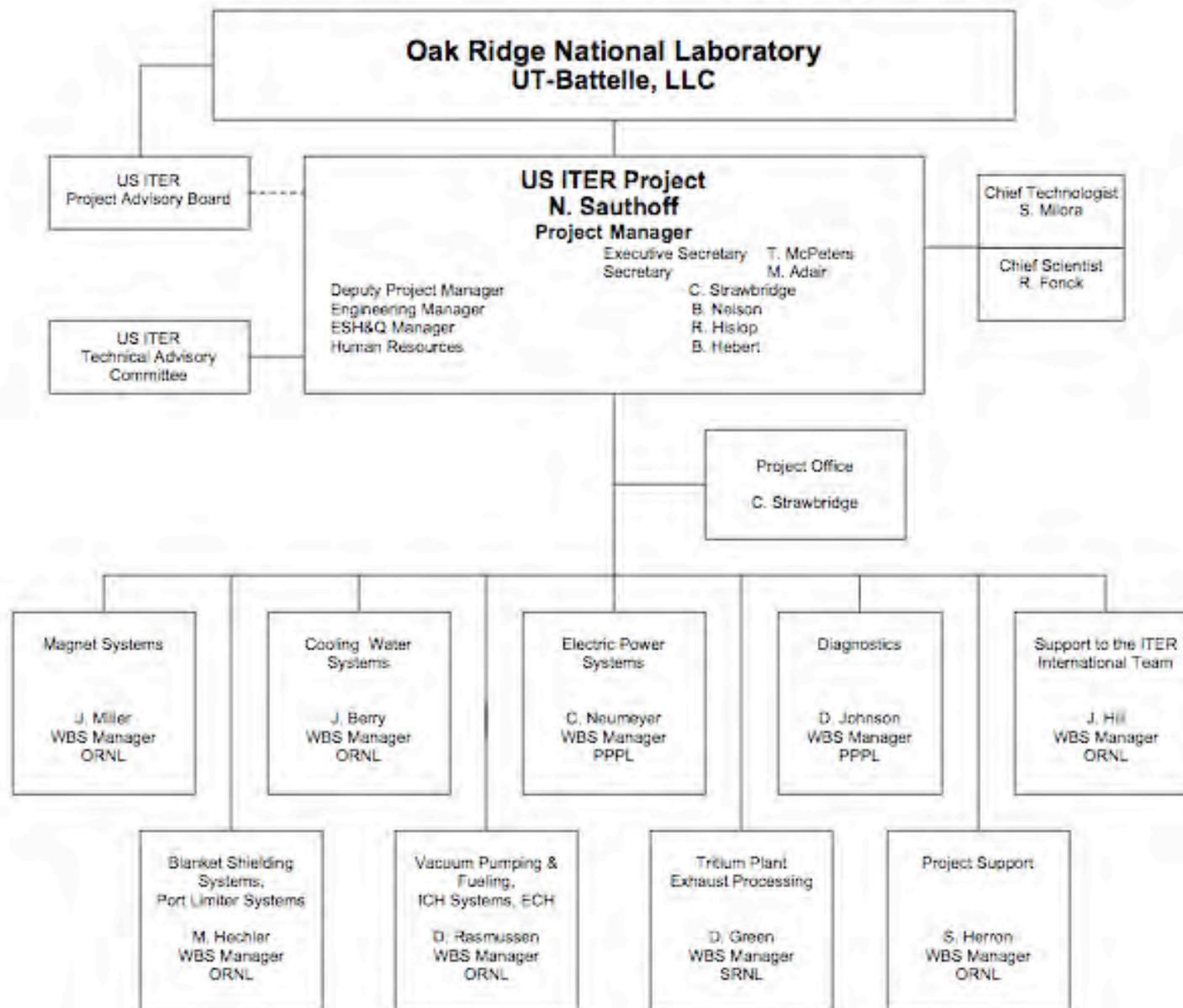
# Highest Level Management Structure



# The US ITER Project Office (US IPO) is established



**The US IPO management team has been fully staffed, with all seven WBS Managers now on board”**



# Support of the International Team

---

- **Cash, as part of “Staff and Infrastructure”**
- **Domestic Staff support of the IT**
  - Design and facilitation of systems with US-scope
  - Project management expert support
- **Candidates to fill “Urgent Positions”**

# IT-selected US Secondee Candidates and Visiting Researchers

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<b>DDG/Tokamak:</b>	<b>Gary Johnson (ORNL)</b>
<b>Buildings:</b>	<b>Jerry Sovka (ATI)</b>
<b>Diagnostics/in-vessel [VR]:</b>	<b>Doug Loesser (PPPL)</b>
<b>Fusion Nuclear Safety:</b>	<b>Dennis Baker (SRNL)</b>
<b>IC H&amp;CD [VR]:</b>	<b>Richard Goulding (ORNL)</b>
<b>IC H&amp;CD:</b>	<b>Joe Tooker (General Atomics)</b>
<b>Magnets [VR]:</b>	<b>Nicolai Martovetsky (LLNL)</b>
<b>Magnets:</b>	<b>Remy Gallix (General Atomics)</b>
<b>Planning:</b>	<b>Larry Lew (High Bridge)</b>
<b>QA:</b>	<b>Ken Sowder (INL)</b>
<b>Site &amp; Facility:</b>	<b>Paul Holik (ORNL)</b>
<b>Vacuum Vessel:</b>	<b>Chang Jun (PPPL)</b>

# Future IT/IO staff selections

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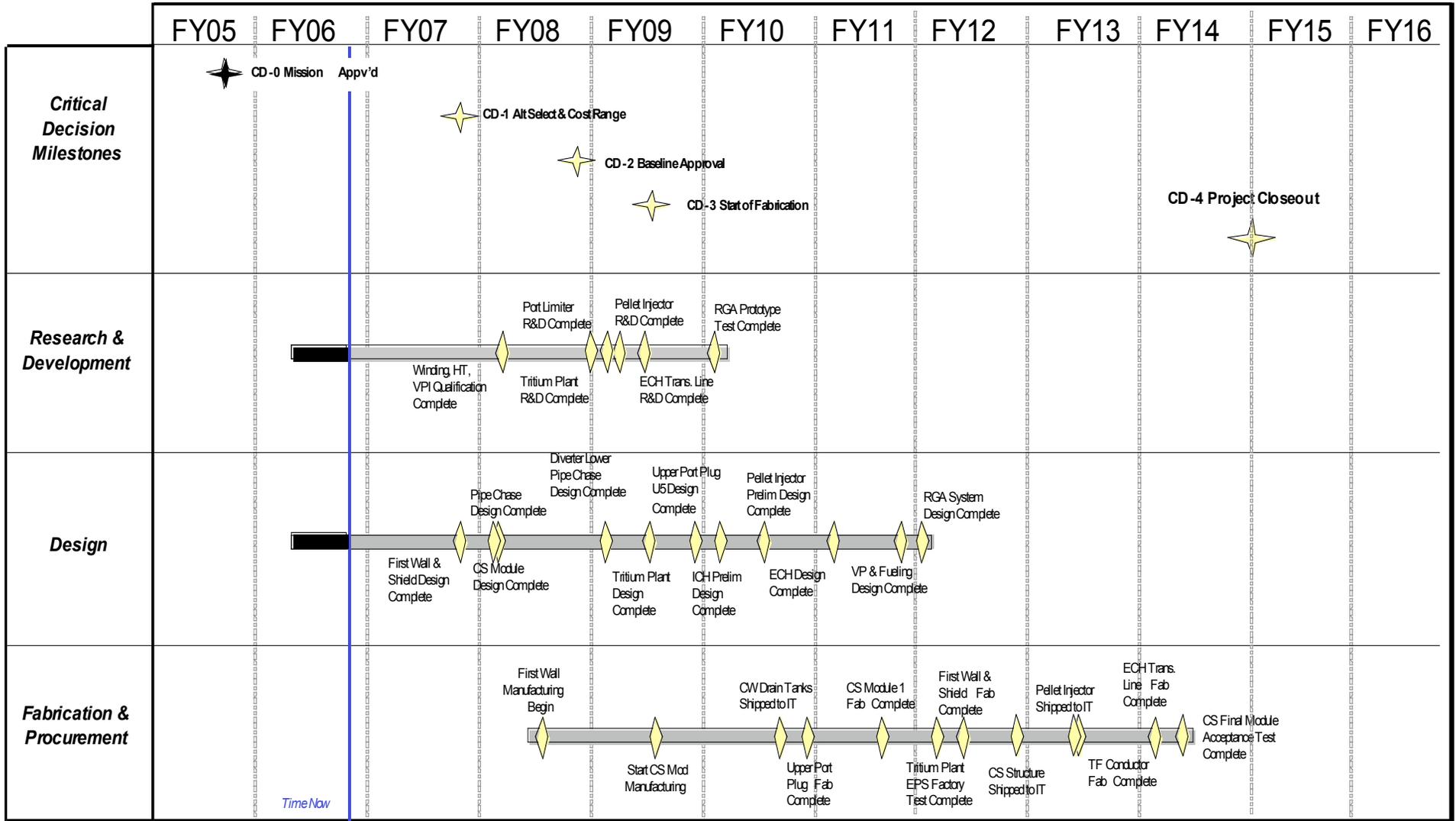
- **Now posted in the US:**
  - Diagnostics Engineer
  - Diagnostic Physicist
  - Superconductor Engineer
  - Coil Designer
- **Expected soon:**
  - ~50 additional positions in physics, technical, administrative, and project management areas

# International Design Review

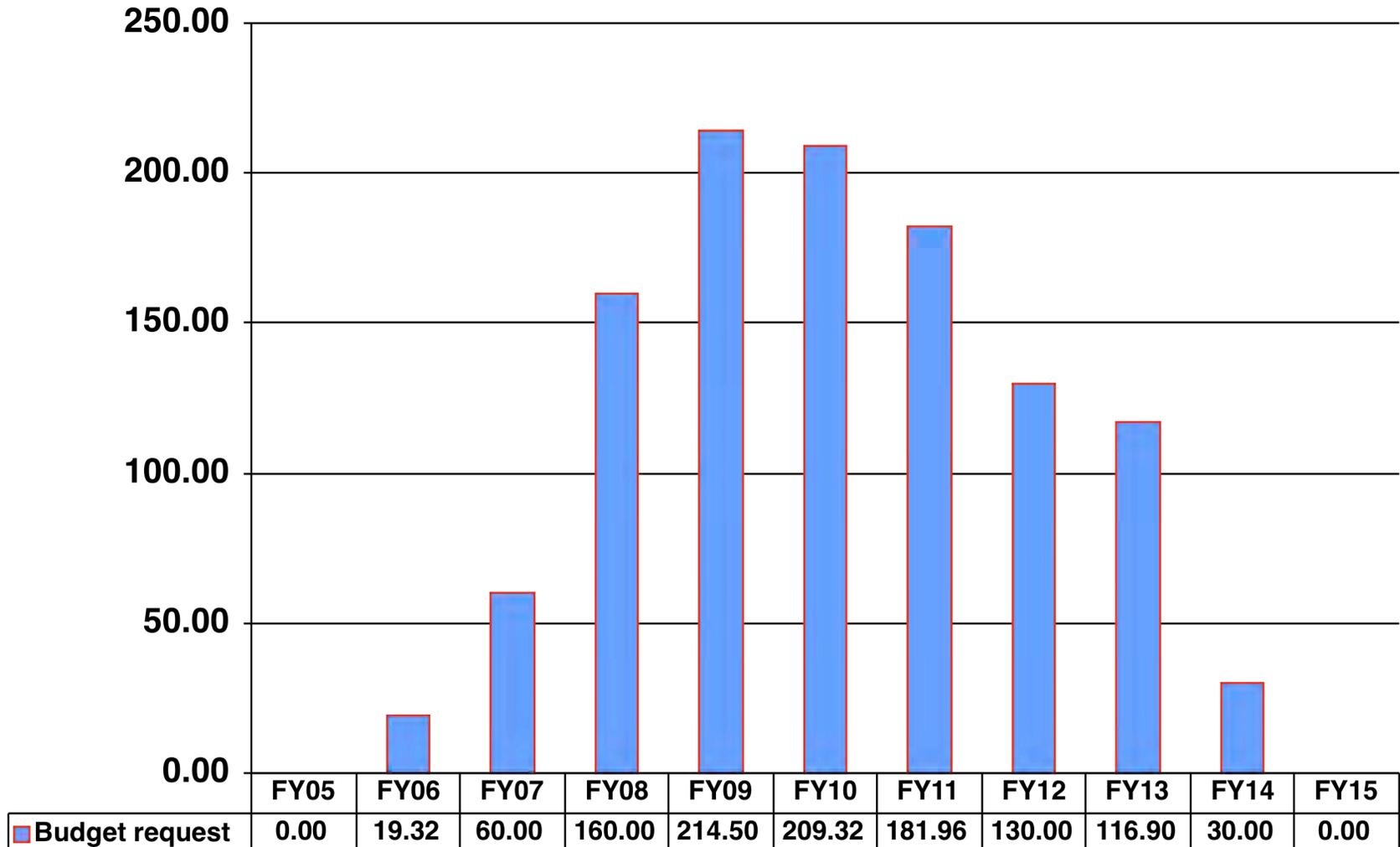
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- **To enable US domestic project progress, the US seeks a revised baseline for the US scope by Summer 2007**
- **Schedule for the Design Review**
  - Now: Preparation, submission and prioritization of “issues;” development of design approaches/solutions
  - Winter 2006/7: International Team Design Review meeting
  - Winter/Spring 2007: International Team, Working Groups and Participant Teams update the design and schedule
  - Spring/Summer 2007: revised Design Baseline
  - ??? 2007: Council-approved Design Baseline

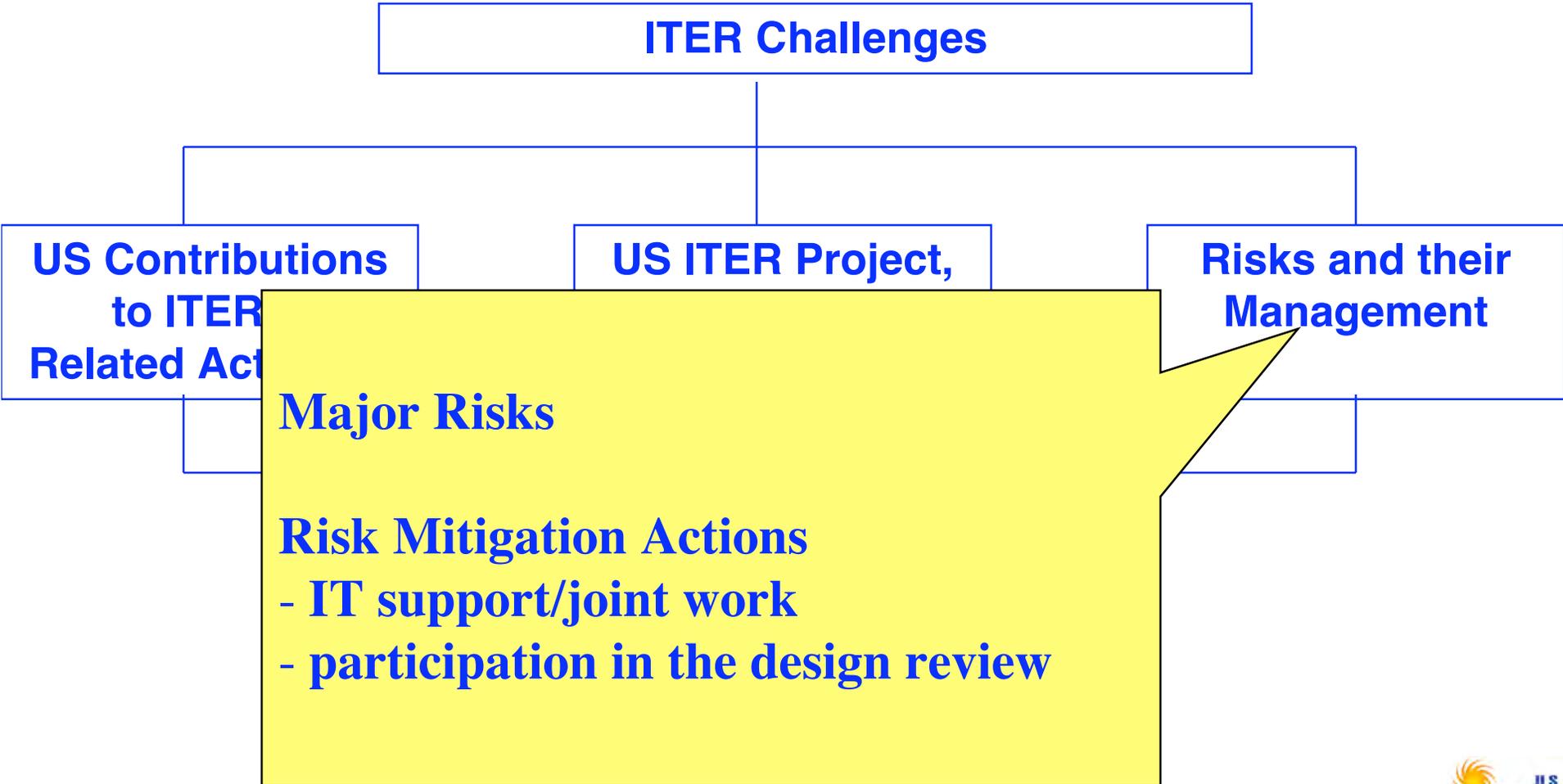
# US ITER Preliminary Schedule



# US ITER Budget Request (\$M), summing to \$1.122B



# Structure of the Talk...



# Major Risks and Issues

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- **Uncertainties about International “need dates” for US components**
- **Uncertainties about contents and availabilities of Internationally-provided component specifications and requirements**
  - Design Baseline
  - Procurement Agreements (technical and management aspects for US procurements)
  - Codes, standards, and Host regulations
- **Uncertainties about drivers of US cost linked to other parties**
  - Sub-components from other parties
  - Responsibilities for costs related to regulatory requirements imposed by the Host

# Summary assessment and path forward

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- **A strong international ITER Organization is key to ITER success**
  - the central ITER team is growing in strength, improving its ability to perform necessary system integration and project leadership
- **The US Domestic Agency senior management is assembled, is building its team and tools, and is developing its FY2007 Work Plans**
- **US Near-term Activities focus on:**
  - Completing R&D, design, and re-baselining of the US scope
    - Positioning for updated scope, and firmer cost and schedule estimates by the end of 2007
    - Supporting the International Team with secondees, on-site experts, and domestic work
  - Enabling the US scientific and technology communities to participate in the upcoming ITER Design Review to make ITER as good a research tool as it can be....