

DE LA RECHERCHE À L'INDUSTRIE

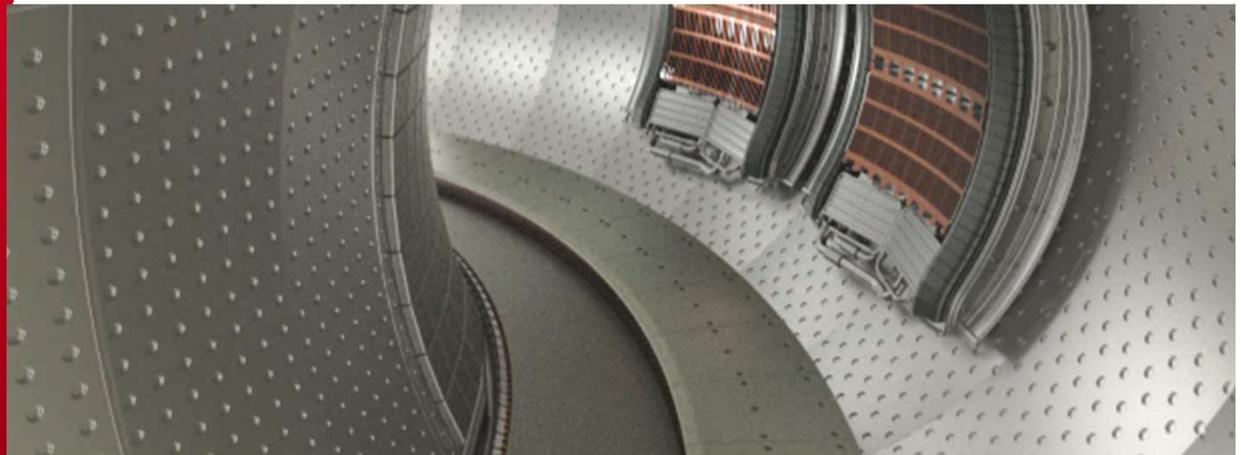
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# WEST PROJECT AND OPPORTUNITIES FOR US- DOE COLLABORATIONS

WEST : W Environment in Steady state Tokamak

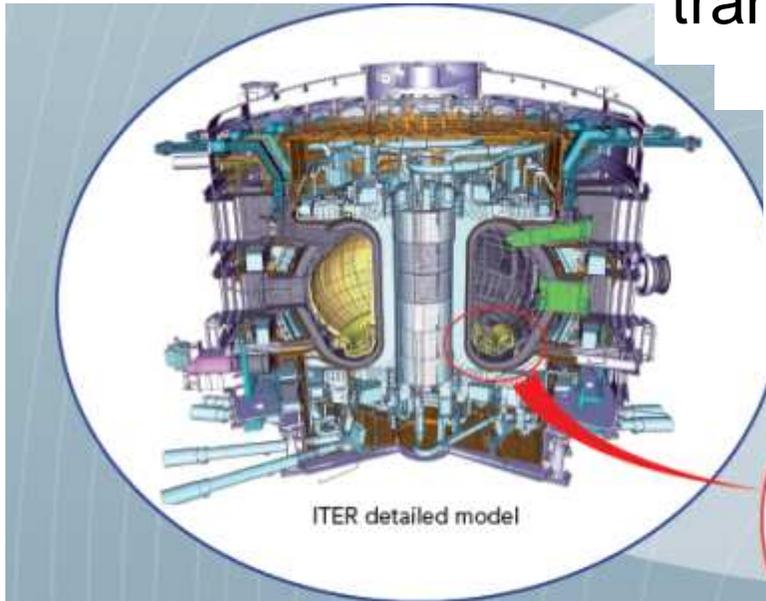


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Project Scientist : E. Tsiatrone together with  
Ph Ghendrih and C. Bourdelle

2014 FESAC Strategic Planning (SP) Panel 8-10 July , USA

# Power exhaust : a challenge for fusion, in ITER and DEMO

- **ITER divertor** : steady state up to 10 MW/m<sup>2</sup> and slow transients up to 20 MW/m<sup>2</sup> + lifetime issue: high particle fluence, erosion, potential melting, ELMs, disruptions, etc



**ITER Divertor**



## Keys figures for ITER divertor risk analysis

- Cost > 100 M€
- Manufacturing: ~ 6 to 8 years
- Installation and commissioning in nuclear environment : ~1 year

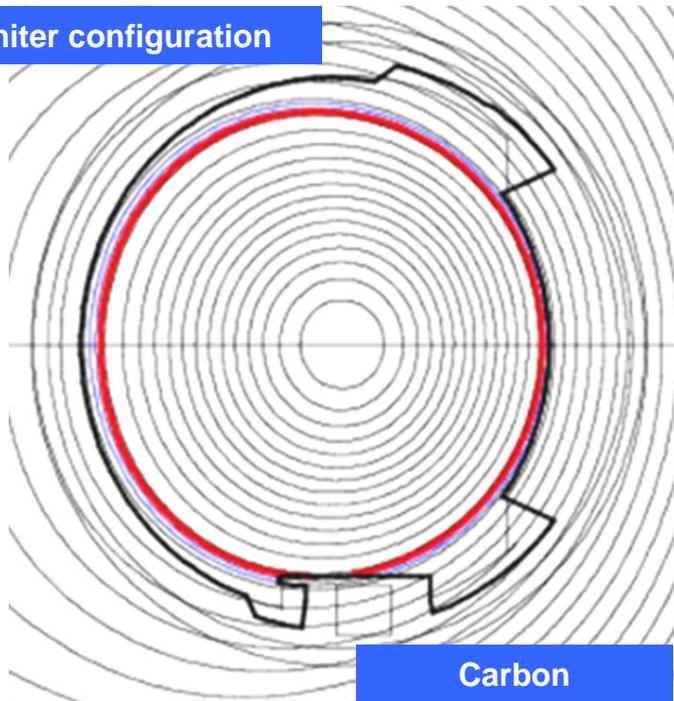
- **WEST** : risk minimisation for **ITER divertor** and explore innovative PFC for DEMO

- Long pulse operations pioneered for more than 20 years (first plasma 1988)
- World record of injected/extracted energy in a tokamak (1GJ)
- Several generations of carbon PFCs designed, manufactured and operated
- **Tore Supra designed for long pulse operation**
  - Superconducting toroidal coils
  - Cryogenic plant
  - Pressurized water loops
  - 15 MW of HF plasma heating
  - Fuelling systems
  - Diagnostics

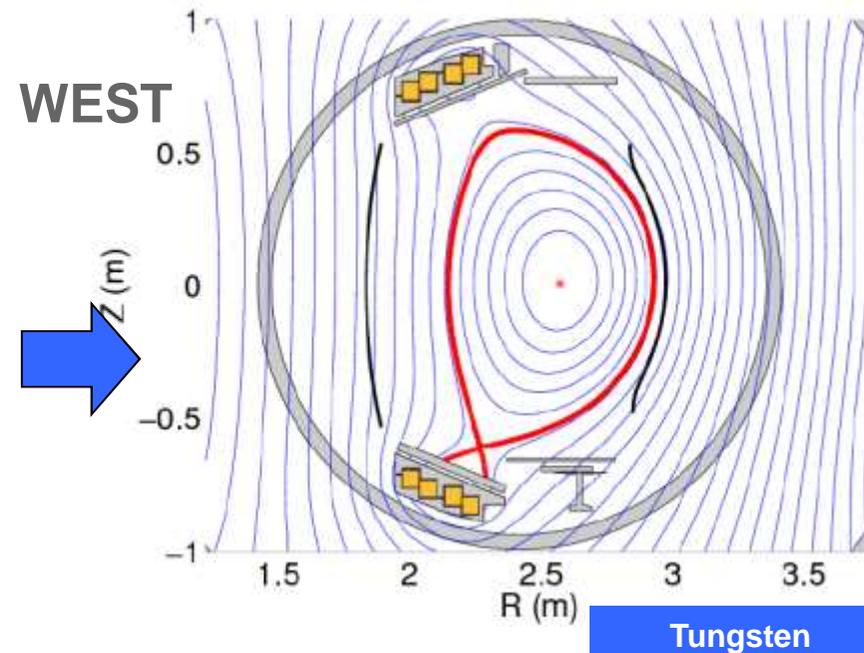


## WEST project ~ few days of ITER operation

Limiter configuration



Divertor configuration



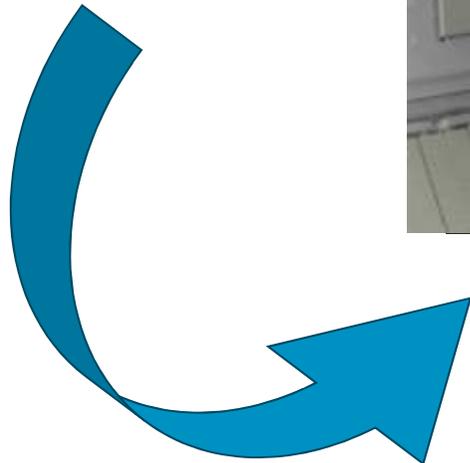
- Two symmetric divertor coils and supporting structures
- Plasma facing components: ITER-like targets, Upper divertor targets, Pumping Baffle, Bumpers, Vessel protection
- 3 new ICRH CW ELMs resilient antennas, 2 LHCD antennas
- Diagnostics: Magnetics, IR , spectroscopy, Langmuir probes etc

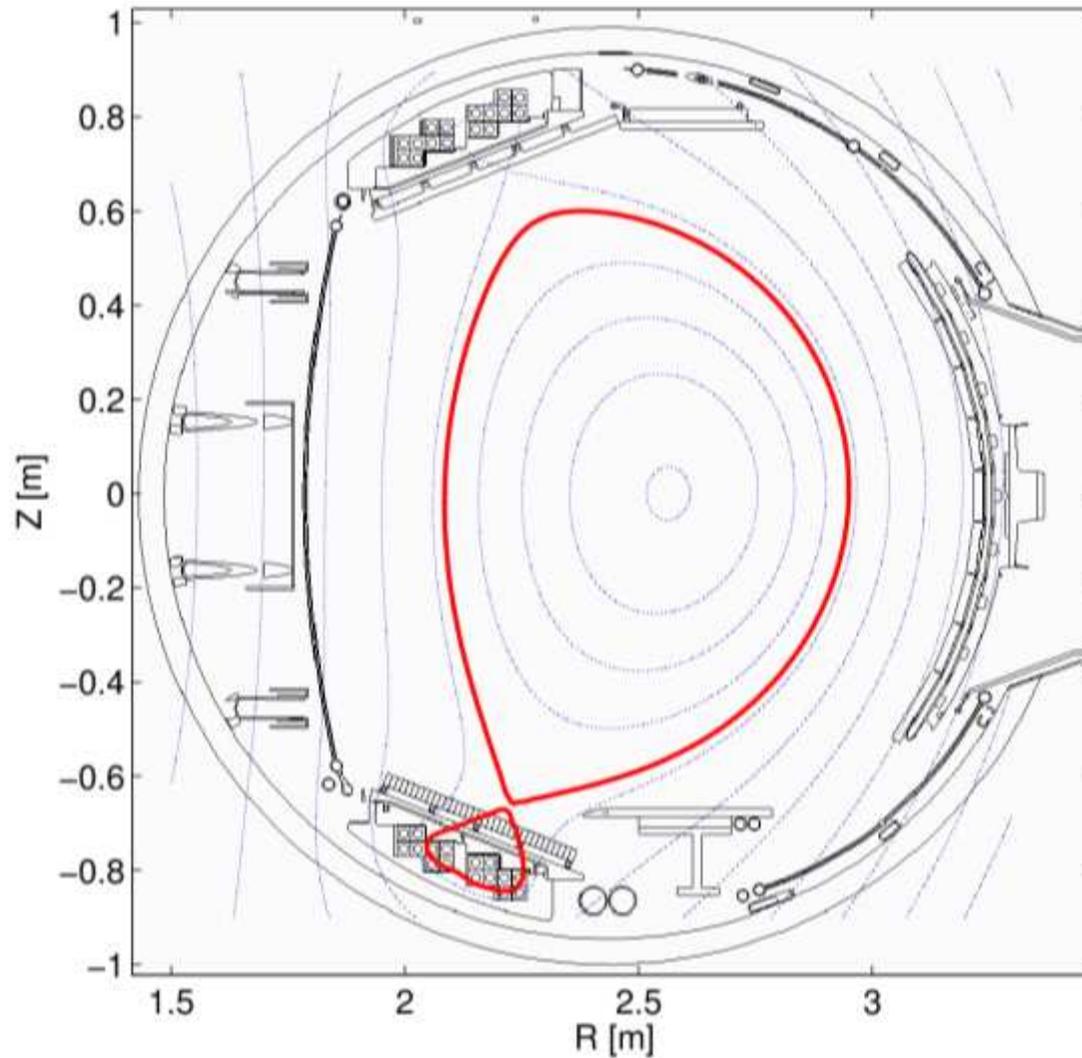


**CIEL configuration**



**WEST configuration**





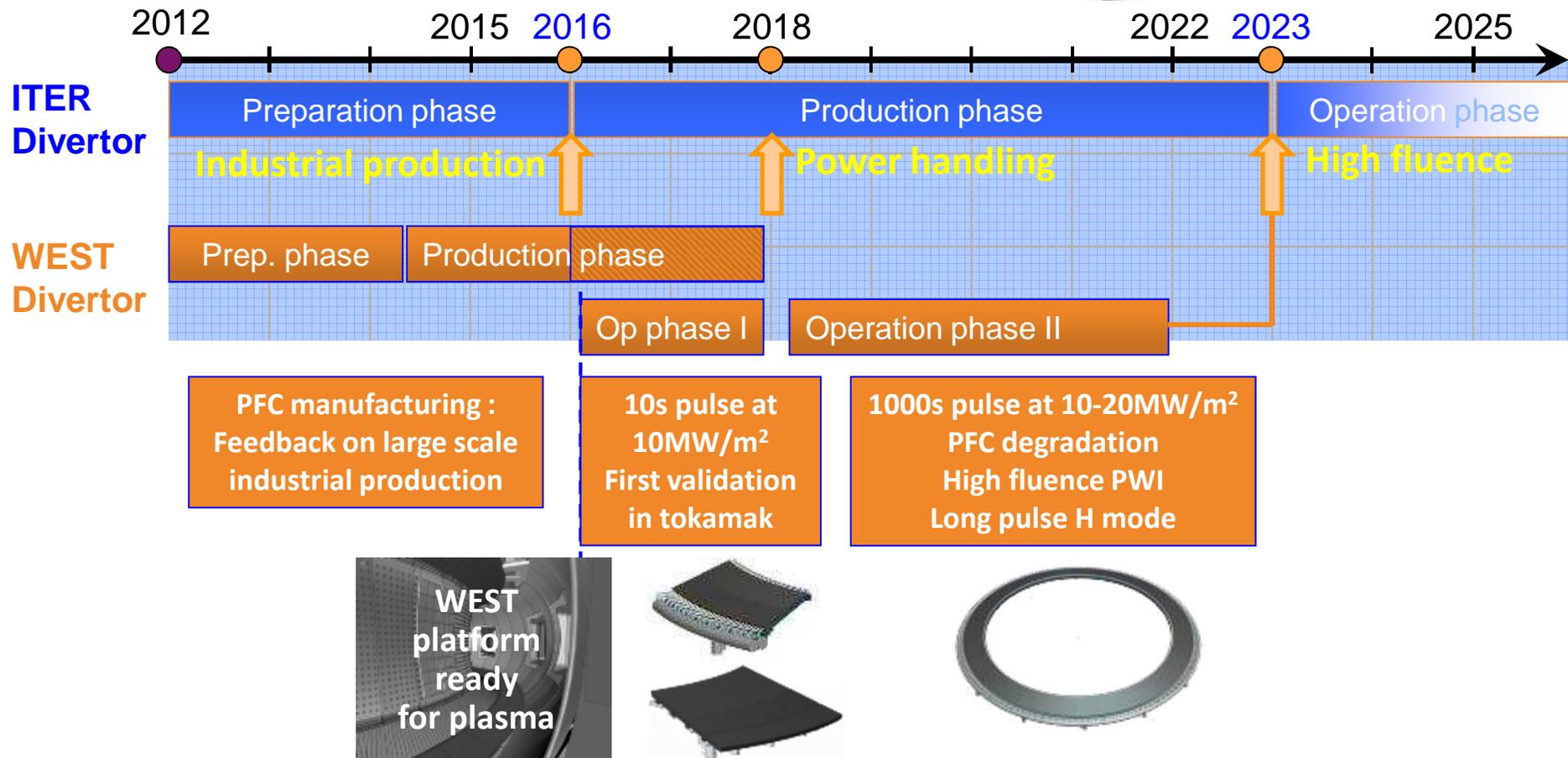
$I_p$ ( $q_{95} \sim 2.5$ )	1 MA
$B_T$	3.7 T
R	2.5 m
a	0.5 m
A	5-6
k	1.3-1.8
d	0.5-0.6
$V_p$	15 m <sup>3</sup>
$n_{GW}$ (1MA)	$1.5 \cdot 10^{20} \text{m}^{-3}$
$P_{ICRH}$	9 MW
$P_{LHCD}$	7 MW
$P_{ECRH}$	0.6 MW
$T_{flattop}$ (0.8 MA)	1000 s

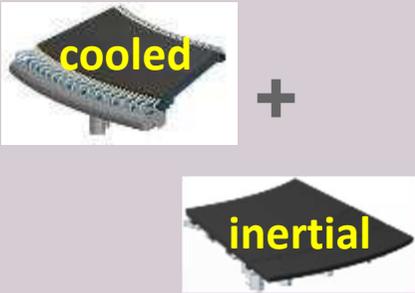
Scenario	High power	High fluence
$I_p$	0.8 MA	0.6 MA
$n_e$	$8 \times 10^{19} \text{m}^{-3}$	$7 \times 10^{19} \text{m}^{-3}$
$f_{GW}$	70 %	70 %
$P_{heat}$	15 MW	10 MW
LHCD	6 MW	7 MW
ICRH	9 MW	3 MW
$W_{th}$	0.9 MJ	0.5 MJ
Bootstrap fraction	30%	35%
LHCD fraction	30%	60%
<b>Pulse length</b>	<b>30 s</b>	<b>1000 s</b>
<b>Expected heat load</b> 2/3 vs 1/3 asym.	<b>10 to 20 MW/m<sup>2</sup></b> depending on X point height and $\lambda_q$	
<b>Operation time to reach one ITER pulse fluence</b> ( $10^{27} \text{D/m}^2$ )	~ 6 months	<b>a few days</b>
$f_{ELM}$	~ 50-400 Hz	
ELM load	up to ~100kJ/m <sup>2</sup>	

# WEST : bringing answers in time for ITER divertor

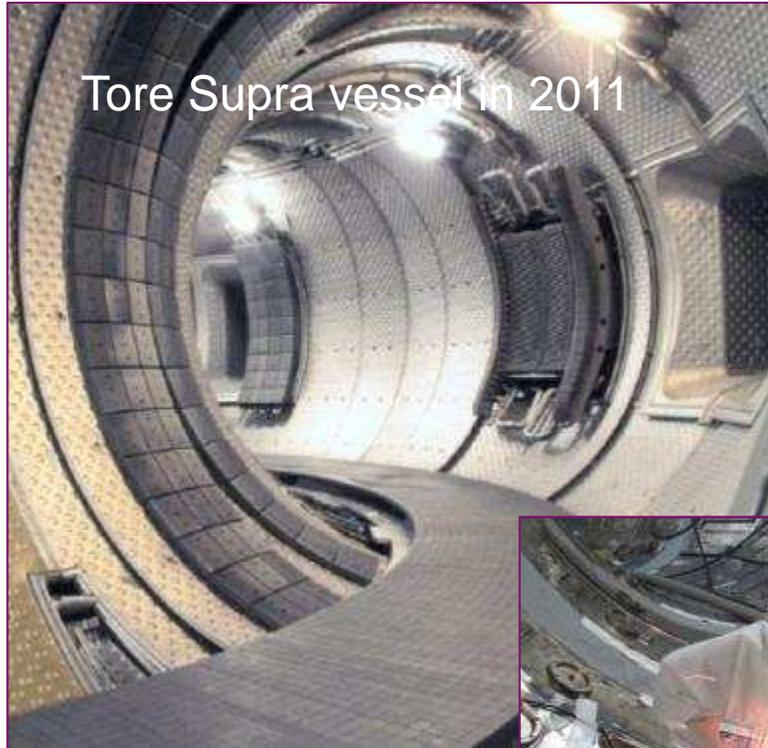


ITER Divertor Installation



	Phase 1	Phase 2	Phase 3
WEST divertor	<p>Mix of actively cooled ITER like / inertial divertor sectors</p> 	<p>Full actively cooled ITER like divertor</p> 	<p>Innovative PFC concepts (tbd depending on outcome of EUROfusion R&amp;D)</p> 
Heating power	Full power	Full power	Full power
Plasma duration	~10 s	Up to 1000 s	Up to 1000 s
Main focus for divertor testing	ITER div 1 / Power handling	ITER div 1 / High fluence PWI	ITER div 2 / DEMO

# TORE SUPRA READY FOR NEW COMPONENTS INTEGRATION: FIRST PLASMA EXPECTED IN 2016



**Toward WEST  
configuration**



## ■ Pave the way towards the ITER actively cooled tungsten divertor procurement and operation

- Optimization of industrial scale production / qualification processes ahead of ITER divertor procurement
- Assessment of power handling capabilities / lifetime of ITER high heat flux tungsten divertor components in tokamak environment
- Validated scheme for protection of actively cooled metallic plasma facing components

## ■ Master integrated plasma scenario over relevant plasma wall equilibrium time scale in a metallic environment

- Demonstration of integrated long pulse H mode scenario (optimization of RF heating, control of tungsten contamination and plasma density)
- Investigation of advanced scenario regimes (fully non inductive H mode operation, highly radiating scenario, ...)

## ■ H1 : testing ITER tungsten PFCs

Objective : 10-20 MW/m<sup>2</sup> on PFC

Questions : Design of leading edges ?

Performance under combined heat+plasma loads ?

Monitoring of metallic PFCs ?

## ■ H2 : towards long pulse H mode operation

Objective : H mode >100 s

Questions : Integrated scenario for long pulse H mode ?

Impurity transport (tungsten accumulation) ?

## ■ H3 : exploring PWI at high fluence

Objective : ITER particle fluence(10<sup>27</sup> D/m<sup>2</sup>)

Questions : Evolution of tungsten surfaces (D/He) ?

Fuel retention in tungsten at high fluence ?

## ■ H4 : preparing advanced tokamak modes

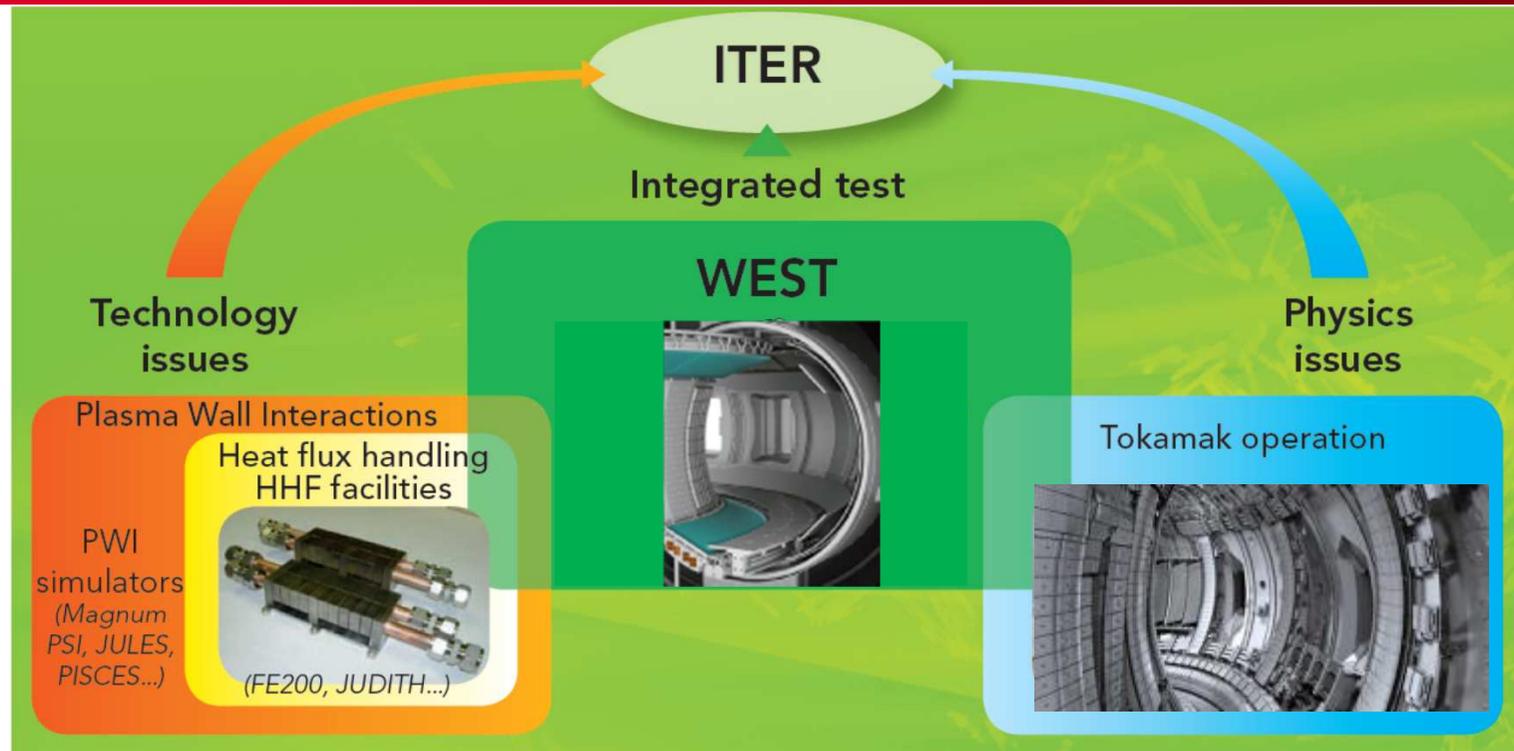
Objective : high power / high bootstrap

Questions : Current profile control for long pulses ?

Hybrid scenario ?

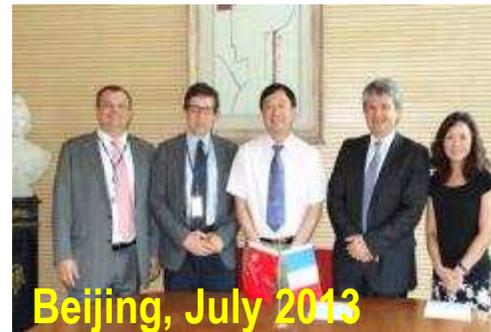
ITER

JT60-SA



- **WEST complementary** to PWI simulators (Magnum PSI, PISCES, JULES,...) and to heat flux handling facilities (FE200, JUDITH, GLADIS,...)
- **WEST complementary** to metallic tokamaks (JET, AUG, EAST, Cmod, FTU, ...) and to steady state programmes (W7X, JT60-SA, LHD, SST-1, HL-2M,...)

- ☐ Jan 2013: China National Nuclear Corporation (associated lab)
- ☐ Jul 2013:
  - Chinese Academy of Sciences (associated lab)
  - India Department of Atomic Energy
  - IPPLM Poland
- ☐ Sept 2013:
  - US-DOE
- ☐ Oct 2013:
  - Fusion For Energy (Europe)
- ☐ Nov 2013:
  - JADA (Japon)
- ☐ Dec 2013:
  - South Korea



**1985:** Cooperation established on long pulse operation in Tore Supra (very strong partnership with ORNL on heating, fueling, particle control, diagnostics)

**2013:** Signature of Letter of Intend. CEA-DSM and DOE-Fusion Energy Science intend to continue and extend their collaboration on:

- Actively cooled metallic plasma facing components
- Operation of long-duration plasma in actively cooled metallic environment
- Preparing ITER generation

**In addition 2011-2014:** Collaboration agreement between CEA & MIT on LHCD, ICRH, intrinsic plasma rotation, diagnostics

## WEST actively cooled metallic plasma facing components

- Operational assessment of ITER W divertor design & technology
- Tungsten erosion, PFC lifetimes, plasma-materials interaction at high power, high fluence plasma
- SOL characterization and modelling

Research programs on PMI/PFC of WEST and MPEX (Material Plasma Exposure eXperiment) would provide a good synergy for integrated solutions suitable for long-pulse operation

## Operation of long duration plasma in actively cooled environment

- Diagnostics plasma core and plasma wall interactions studies and protection of plasma facing components
- Development of long-pulse RF wave heating systems
- Development of safe & integrated controlled operation: controlled schemes, disruptions, ripple losses...
- Joint theory & modelling

- According to the EUROfusion proposal some EU facilities exploited as common facilities under campaign-oriented approach.
- EU facilities selected following 2008 Facility Review.
- For possible new facilities to be supported under the same provisions, in the EUROfusion proposal it has been stated:
  - **Should other tokamaks become available, their use under the campaign-oriented approach will be assessed as during the 2008 Facility Review, and, in case of a positive assessment, included within this Work Package**

- Panel in charge of the evaluation of WEST as a EUROfusion facility  
Boris Sharkov (chair) , C. Linsmeier (FZJ), A. Loarte (ITER Org), G. Matthews (CCFE), P. Mertens (FZJ), W. Morris (CCFE), Y. Ueda (University of Osaka), R. Parker (MIT), H. Zohm (IPP), + D. Maisonnier (Commission Observer) + D. Borba (EUROfusion PMU)
  
- The panel evaluates the potential contribution of WEST to the European Fusion Roadmap, the added value and the cost/benefit ratio of WEST compared to the already existing or upcoming EU facilities (JT60-SA) :
  - to address the ITER & DEMO heat exhaust problem (Mission 2),
  - to reduce gaps identified in the Facility Review and the European Fusion Roadmap
  - to minimize risks for ITER construction and operation.
  
- **The panel report/recommendation: an important input for the decision at EUROfusion General Assembly and then for the detailed implementation by the EUROfusion Programme Management Unit**

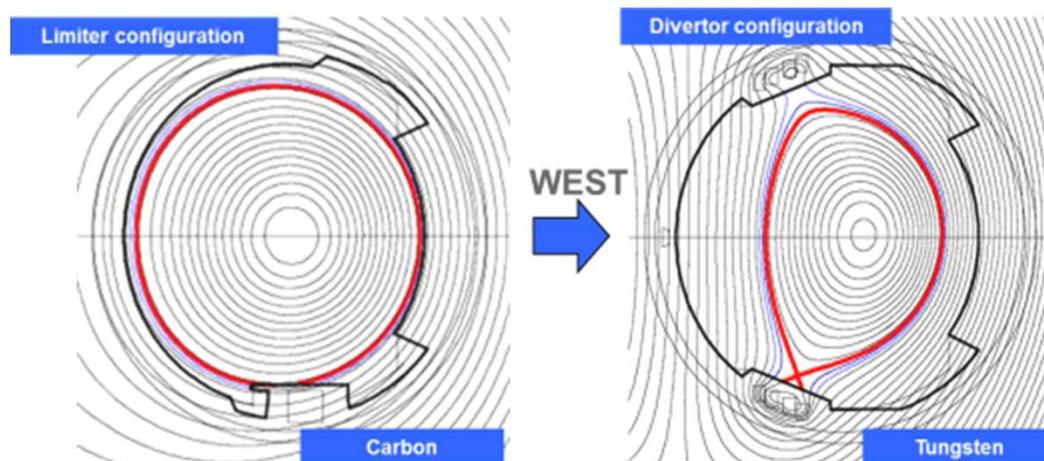
# WEST fills the gap of long pulse operation in the EU program

■ steady state / long pulse physics and technology : key area towards fusion power plant

## Facility Review 2008, gaps analysis for Mission 4 « Physics and technology of long pulse and steady state » :

Gaps exist since TORE SUPRA, the only European device with superconducting coils, cannot access fully ITER relevant scenarios (circular limiter plasma and non-metallic wall)

■ This gap is now filled with WEST.



→ access for EUROfusion to steady state operation / metallic environment

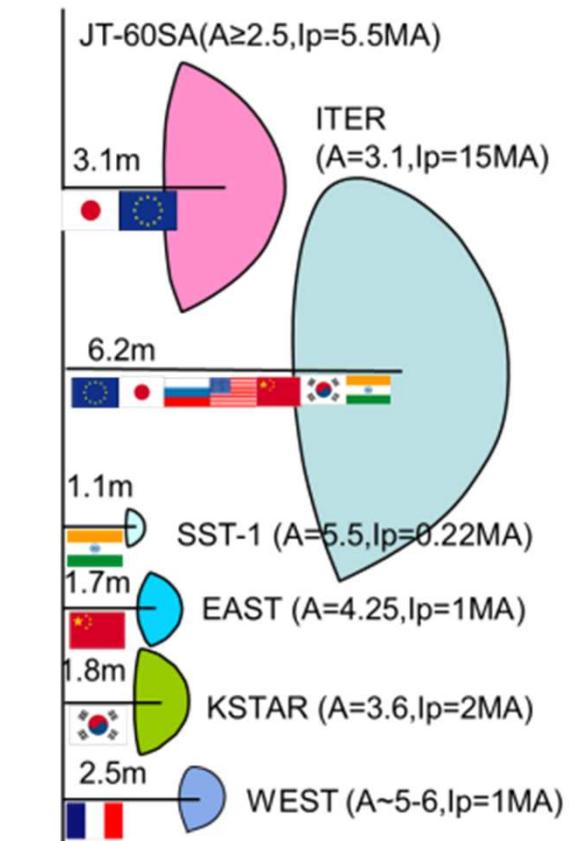


Chart of long pulse superconducting tokamaks

[Courtesy Y. Kamada]

- WEST provides a long pulse facility to prepare actively cooled tungsten divertor operation and train ITER generation
- Research Plan (V0) written
- Modelling of the WEST operational scenario has started
  - sustainment of long pulse H mode with the available power
  - possibility to explore hybrid and steady-state operation
  - address ITPA priorities
- On-going effort with more sophisticated modelling and validation on existing experiments (JET, ASDEX-U, EAST, ALCATOR-C)
  - full CRONOS modelling with LUKE, EVE, TGLF/QuaLiKiz, impurity transport ...
  - ELMs modelling with JOREK
  - W source & W transport modelling (COREDIV, DIVIMP)
  - SOL modelling (SOLEGE2D-Eirene) and coupled core-SOL modelling



## 2014: Consolidating the Research Plan with WEST partners

- WEST international workshop (June 30-July 2, 2014, Aix en Provence)  
<http://west.cea.fr/Workshop2014/>
- V1 of the WEST Research Plan: end 2014

## 2015: Preparing WEST exploitation

- Preparation of the 2016-2017 experimental campaign
- Organization for operation with WEST partners
- WEST platform = user facility open to ITER partners