

The Promise and Status of Compact Stellarators

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Compact Stellarators

NCSX

Promise

- Solve critical problems for MFE.
- Improve on previous stellarator designs.
- Advance fusion science.

Status

- Physics basis for compact stellarator experiments.
- Design.
- Construction of NCSX.

Compact Stellarator Motivation



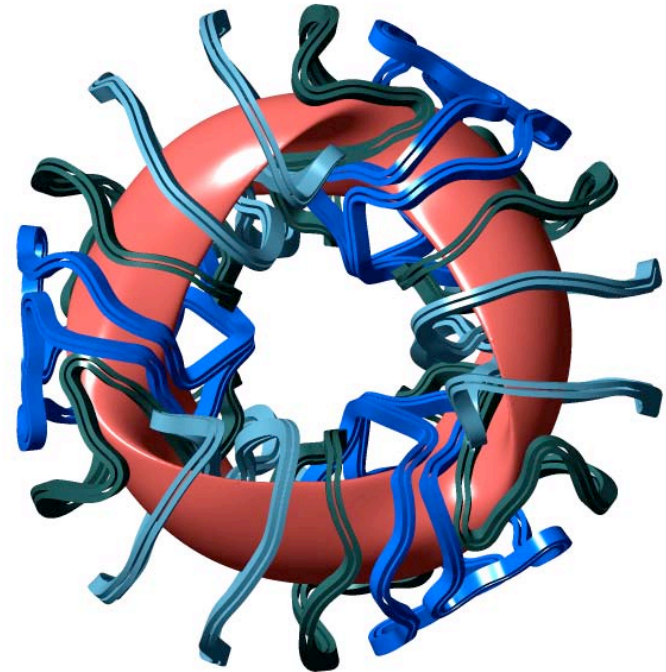
Stellarators solve critical problems for magnetic fusion.

- Steady state without current drive.
- Stable without feedback control or rotation drive. No disruptions.

Compact Stellarators (CS) improve on previous designs.

- Magnetic quasi-symmetry:
 - good confinement.
 - link to tokamak physics.
- Lower aspect ratio.

3D geometry produces benefits and costs. We need to quantify both.

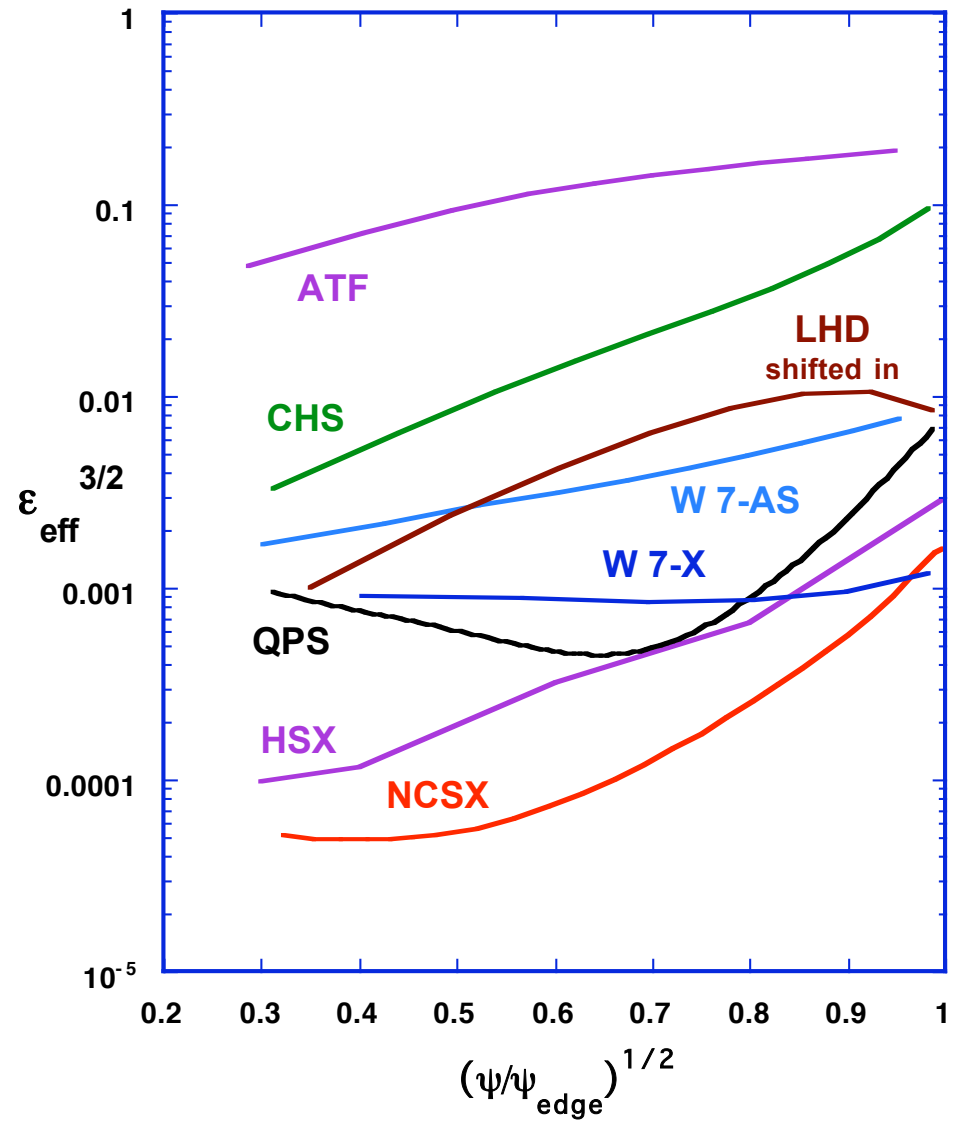


**3-Period NCSX Plasma
and Coil Design**

Quasi-Axisymmetric: Very Low Effective Ripple

NCSX

- In NCSX:
 - $\epsilon_{\text{eff}} \sim 1.4\%$ at edge, $\sim 0.1\%$ in core
- Predicted ripple transport is negligible.
 - neo. transport $\propto \epsilon_{\text{eff}}^{3/2}$ in $1/\nu$ regime.
 - Confinement improves with lower ϵ_{eff} in experiments.
- Gives low flow-damping.
 - allows manipulation of flows for flow-shear stabilization, control of E_r
- Allows balanced-NBI with acceptable losses (24% at 1.2T).



~ Normalized Minor Radius (r/a) **6**

Stellarator Research Advances Fusion Science



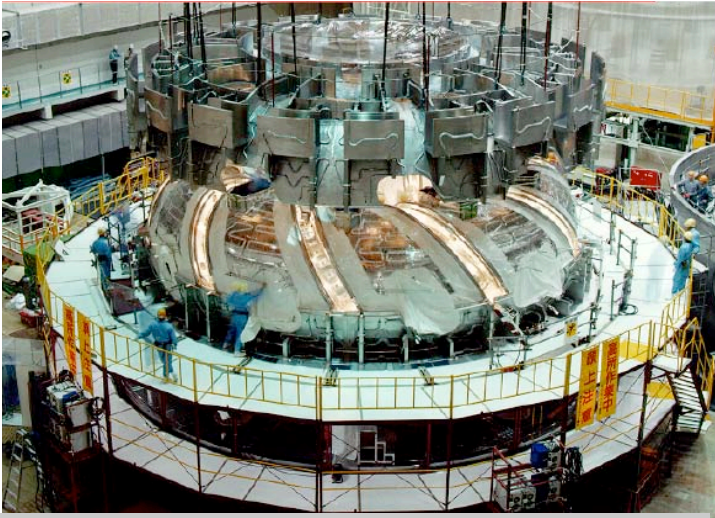
Understanding 3D plasma physics important to all of MFE science

- Rotational transform sources (int., ext.): effect on stability, disruptions?
- 3D plasma shaping: stabilize without conducting walls or feedback?
- Magnetic quasi-symmetry: tokamak-like fundamental transport properties?
- Effects of 3-D fast ion resonant modes & Alfvénic modes in 3-D?
- 3D divertors: effects on boundary plasma, plasma-material interactions?

Answering critical fusion science questions, e.g.

- How does magnetic field structure impact plasma confinement?
 - plasma shaping? internal structure? self-generated currents?
- How much external control *vs.* self-organization will a fusion plasma require?

Stellarators Are Making Excellent Progress



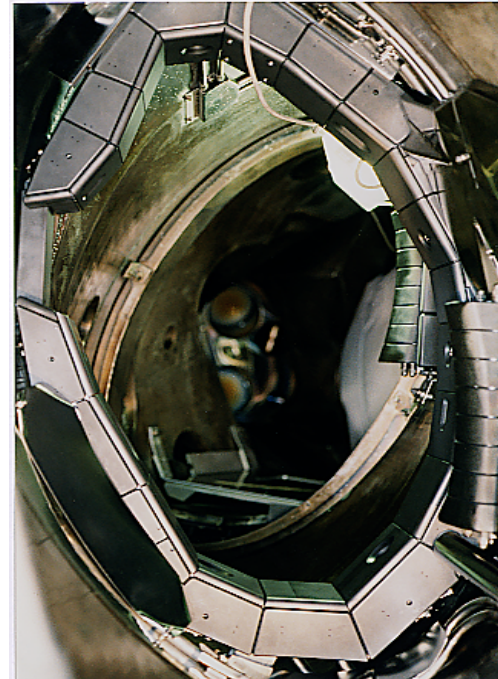
**Large Helical Device
(S/C magnets - Japan)**

$\beta \sim 4\%$.

$T_e \approx 10 \text{ keV}$, $T_i \approx 10 \text{ keV}$.

enhanced confinement.

2-minute pulses.

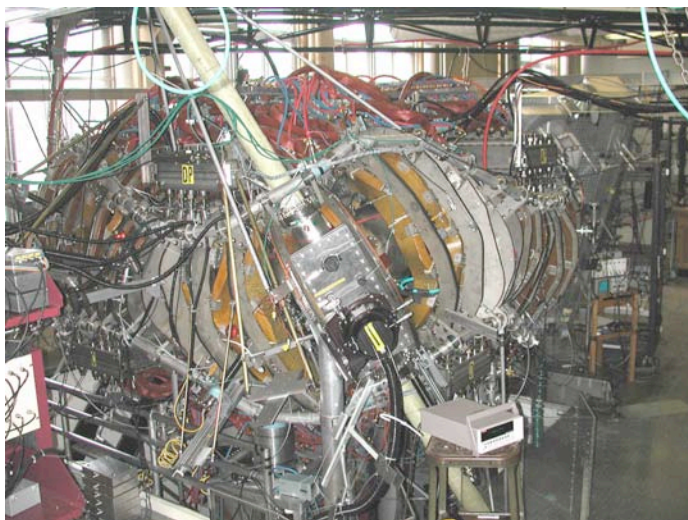


**Wendelstein 7-AS
(Germany)**

$\beta \sim 3.5\%$.

enhanced
confinement.

density control &
enhanced
performance
w/island divertor.



**Helically Symmetric Experiment
(U. Wisc.)**

- Successful test of quasi-symmetry.

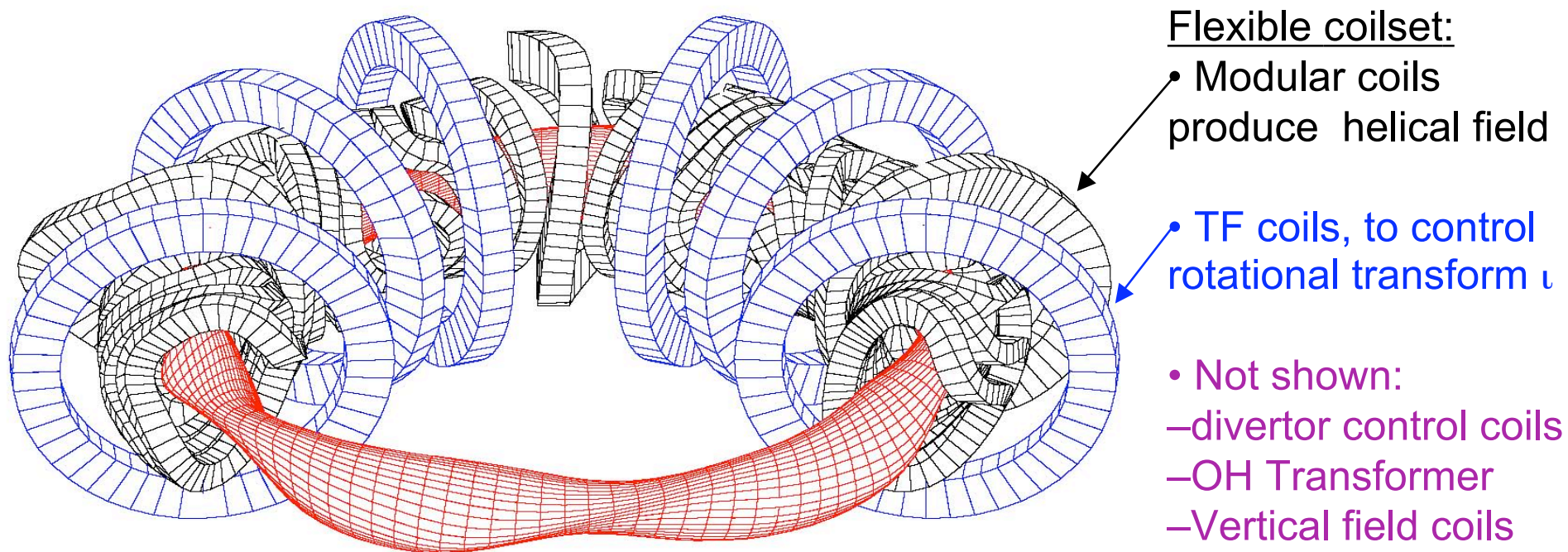
Wendelstein 7-X (Germany)

Optimized Design - S/C magnets

Under construction - Ops. In 2010

W7-AS – a flexible experiment

5 field periods, $R = 2$ m, minor radius $a \leq 0.16$ m, $B \leq 2.5$ T,
vacuum rotational transform $0.25 \leq \iota_{\text{ext}} \leq 0.6$

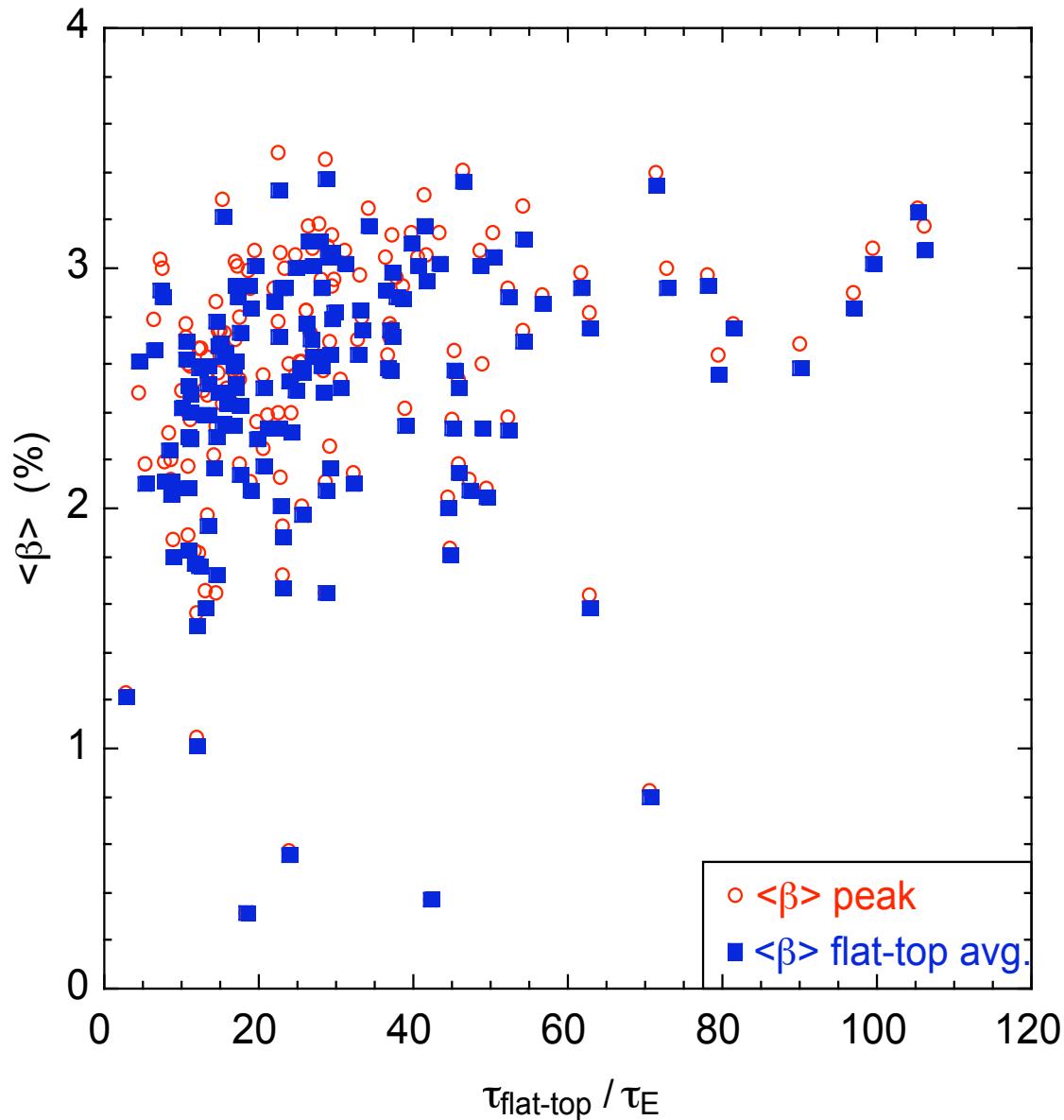


W7-AS

Completed operation in 2002



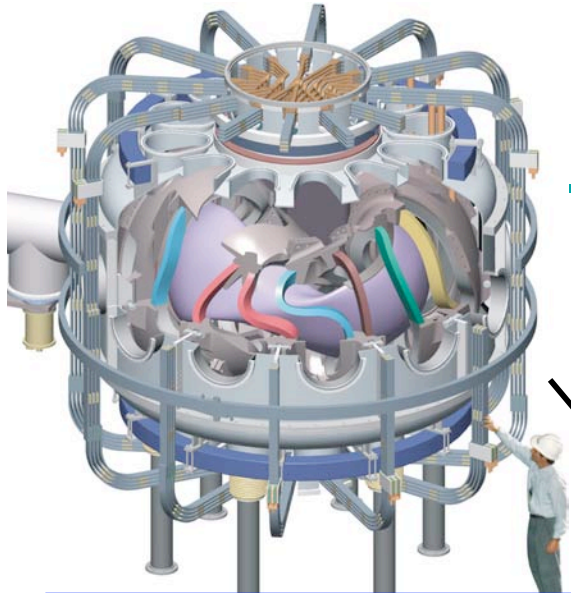
$\langle\beta\rangle > 3.2\%$ maintained for $> 100 \tau_E$ in W7-AS



- Peak $\langle\beta\rangle = 3.5\%$
- $\langle\beta\rangle$ -peak \approx $\langle\beta\rangle$ -flat-top-avg
 \Rightarrow very stationary plasmas
- No disruptions
- Duration and β not limited by onset of observable MHD
- High- β maintained as long as heating maintained, up to power handling limit of PFCs.
- β limit may be set by equilibrium degradation.
 \Rightarrow can avoid by design.

M. Zarnstorff (PPPL) & W7-AS Team.

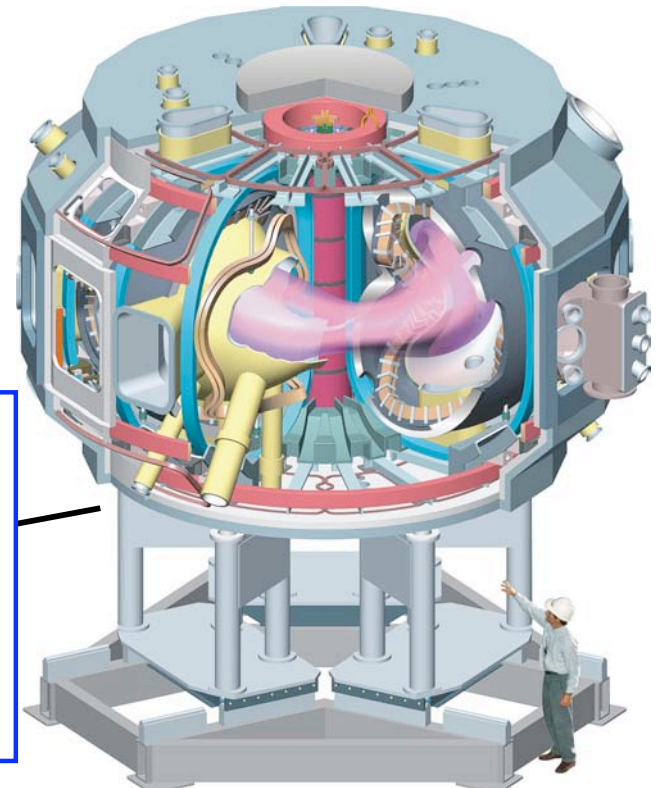
U.S. Stellarator Program Goals: CS Attractiveness, 3D Physics



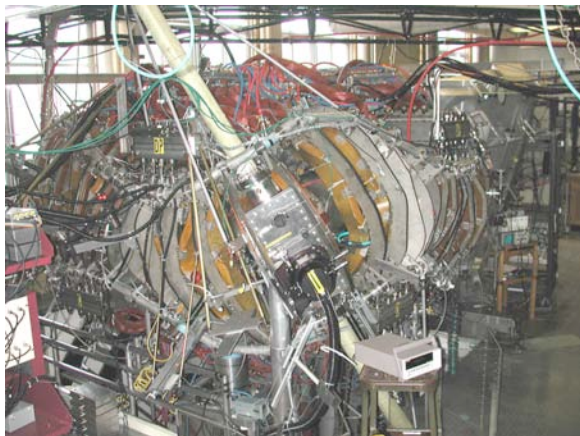
**QPS (ORNL)
CD-1 Approved**

- International Programs (NIFS, IPP,...)
- Theory & Computation
- ARIES-CS Power Plant Study

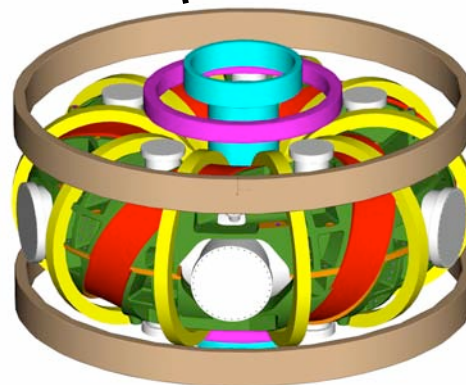
- Test expected CS physics benefits.
- Advance 3D plasma physics.
- Support next-step decisions.



**NCSX (PPPL-ORNL)
Under Construction
First Plasma - 2008**



HSX (U. Wisconsin)



**CTH (Auburn U.)
Ops. in 2005**

NCSX Mission: Develop Physics Basis for Compact Stellarators



Acquire the physics data needed to assess the attractiveness of compact stellarators; advance understanding of 3D fusion science.

Understand...

- Beta limits and limiting mechanisms.
- Effect of 3D magnetic fields on disruptions
- Reduction of neoclassical transport by QA design.
- Confinement scaling; reduction of anomalous transport.
- Equilibrium islands and neoclassical tearing-mode stabilization.
- Power and particle exhaust compatibility w/good core performance.
- Alfvénic mode stability in reversed shear compact stellarator.

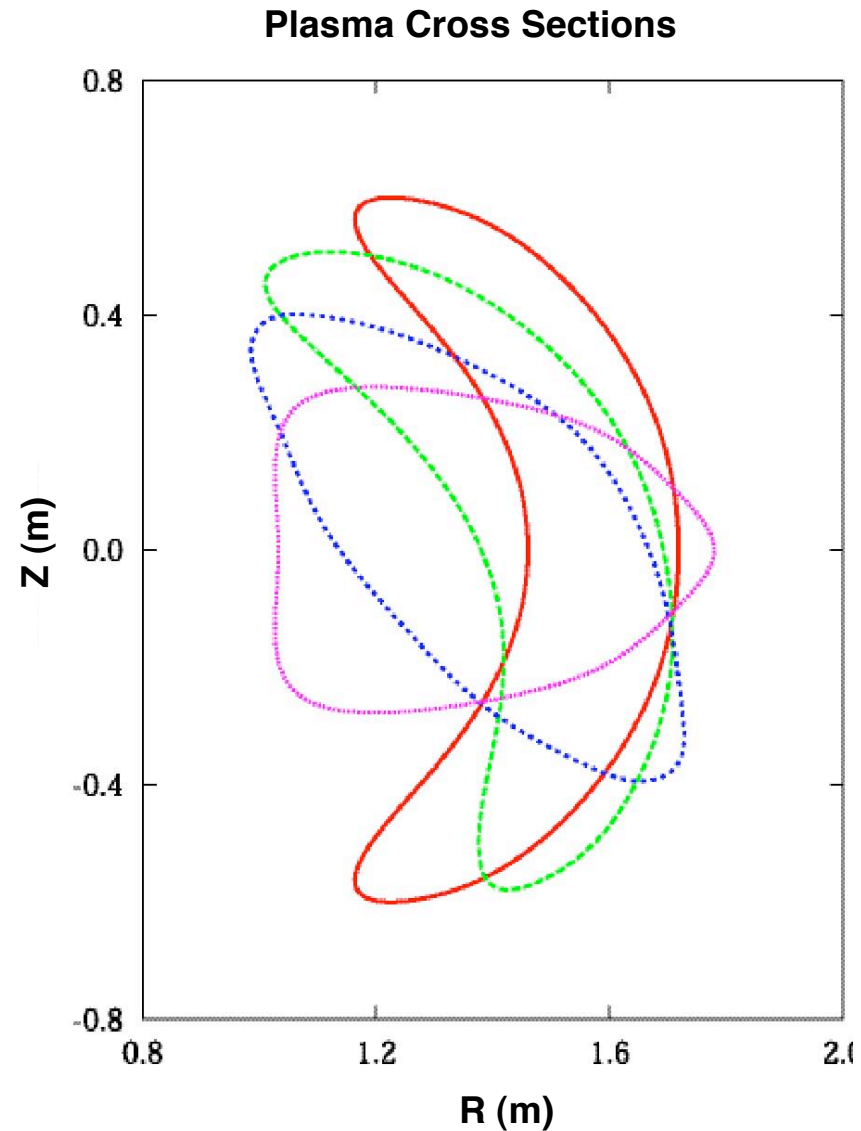
Demonstrate...

- Conditions for high-beta, disruption-free operation.

NCSX Physics Design Target: Attractive Properties

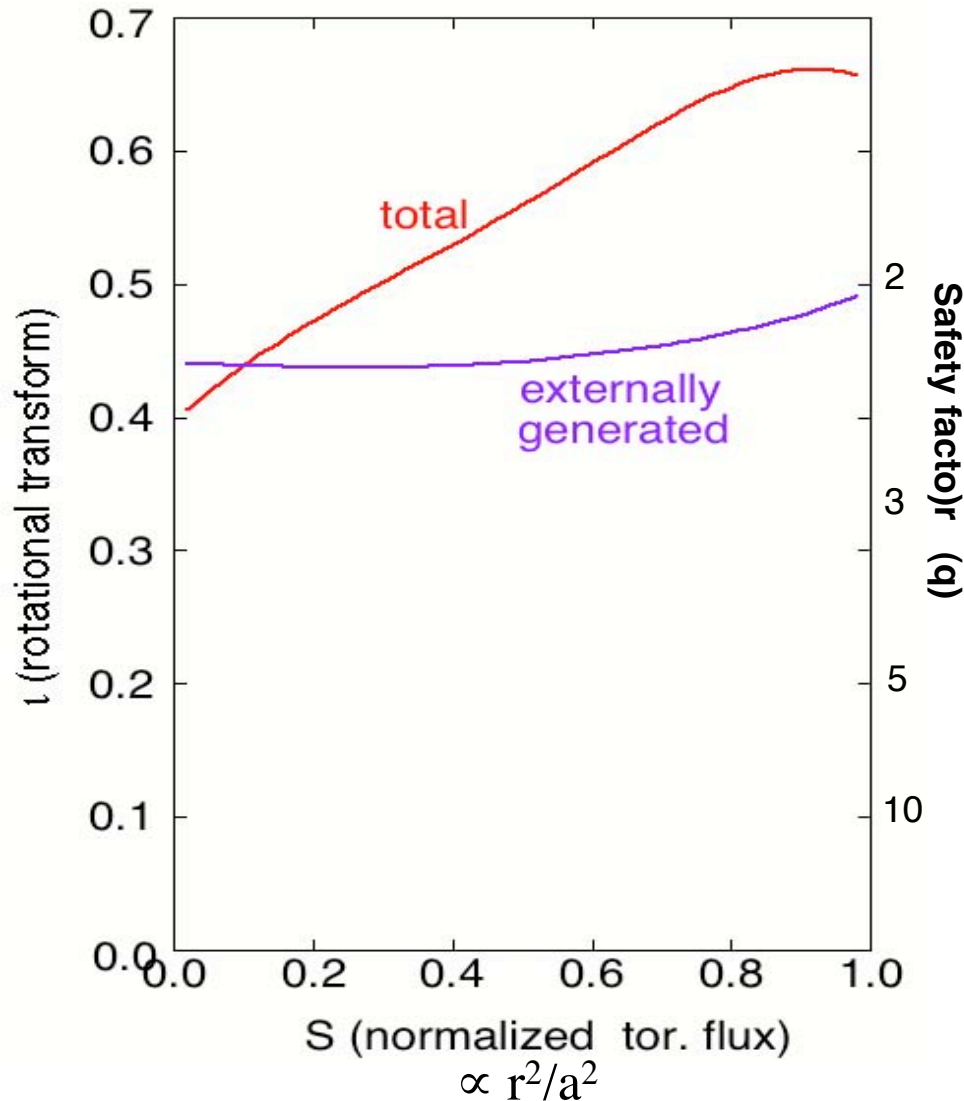


- 3 periods, low $R/\langle a \rangle$ (4.4).
- Quasi-axisymmetric w/ low ripple.
- Stable at $\beta=4.1\%$ to ballooning, kink, vertical, Mercier modes, w/out conducting walls or feedback.



Hybrid Configuration Combines Externally-Generated Fields with Bootstrap Current

NCSSX

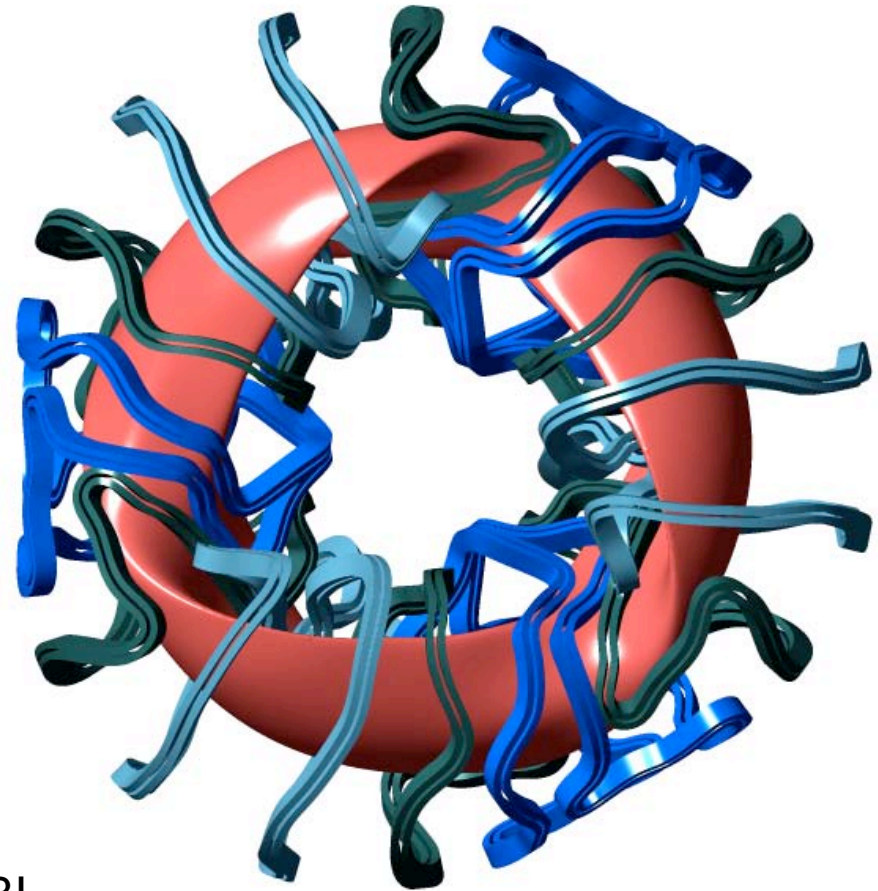


- Assumed moderately broad pressure profile and consistent bootstrap current profile.
- “Reversed shear” iota profile (0.39–0.65).
 - stabilize neoclassical tearing modes.
- $\sim 3/4$ of transform (poloidal-B) from external coils.
- $\sim 1/4$ of transform from bootstrap current.

Coil Design Satisfies Physics and Engineering Criteria

NCSX

- NCSX design uses 18 modular coils (3 shapes)
 - Also TF, PF, and helical trim coils.
- Free-boundary method was used to optimize coils for target properties.
 - VMEC and PIES 3D equilibrium codes.
- Required properties are realized:
 - Free-boundary equilibrium with the required physics properties ($R/\langle a \rangle$, QA, stability at $\beta = 4\%$, iota profile).
 - Engineering feasibility metrics: coil-coil spacing, min. bend radius, tangential NBI access, coil-plasma spacing.
 - Good magnetic surfaces at high β .



**NCSX Plasma
and Modular Coils**

NCSX Coil Design Produces Good Surfaces at High β

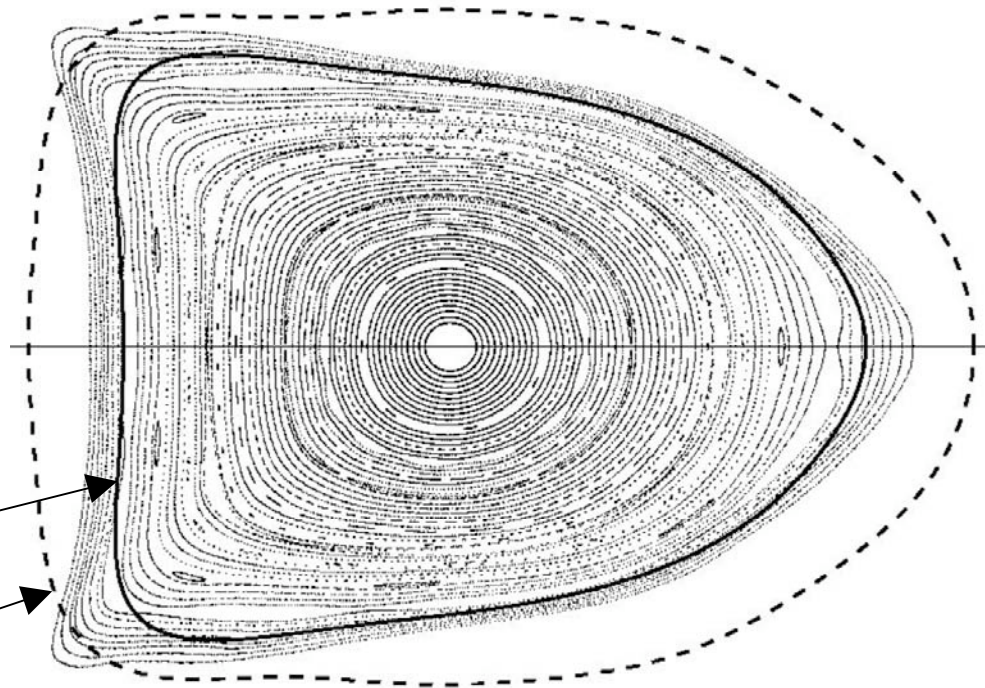


- Coil geometry adjusted to “heal” islands (measured with PIES code) while preserving physics and engineering properties.
- Corrections for neoclassical and finite $\chi_{\perp}/\chi_{\parallel}$ effects (not included in PIES calculation) reduce effective island width by factor 2-3.

- Free-boundary PIES equilibrium for $\beta = 4\%$.
- Multi-filament coils.

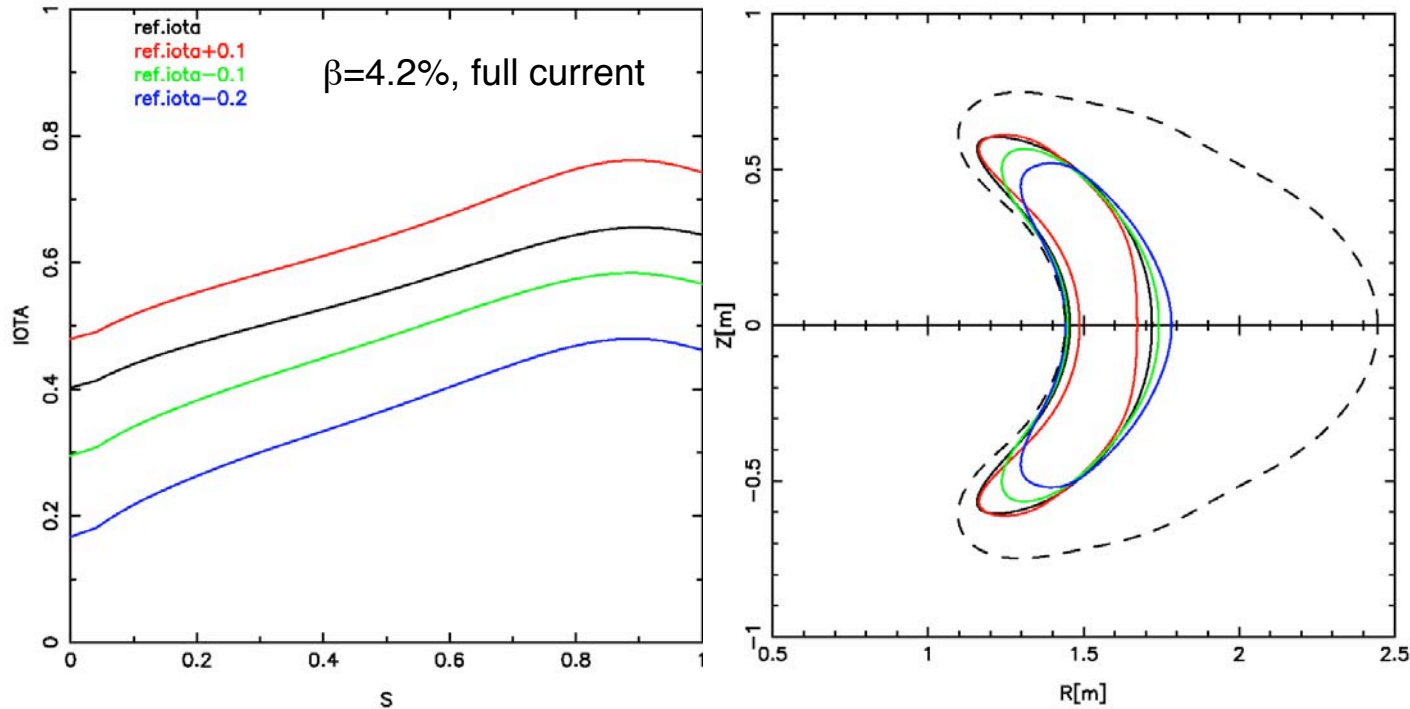
Sum of effective island widths is $<1\%$.

VMEC plasma boundary with unhealed coils
First wall boundary



Also, good surfaces in a range of vacuum configurations.

NCSX Coils: Flexibility to Vary Physics Properties



External iota controlled by plasma shape at fixed profiles.

Also

- Can externally control shear.
- Can increase ripple by $\sim 10x$, preserving stability.
- Can lower theoretical β -limit to 1%.
- Can cover wide operating space in β (to at least 6%), I_p , profile shapes.

NCSX Machine Parameters

NCSX

Stellarator

Major radius: 1.4 m

Performance:

Magnetic Field Strength (B)

@ 0.2 s pulse: 2.0 T

@ 1.7 s pulse: 1.2 T

Vac. base pressure: 2×10^{-8} torr

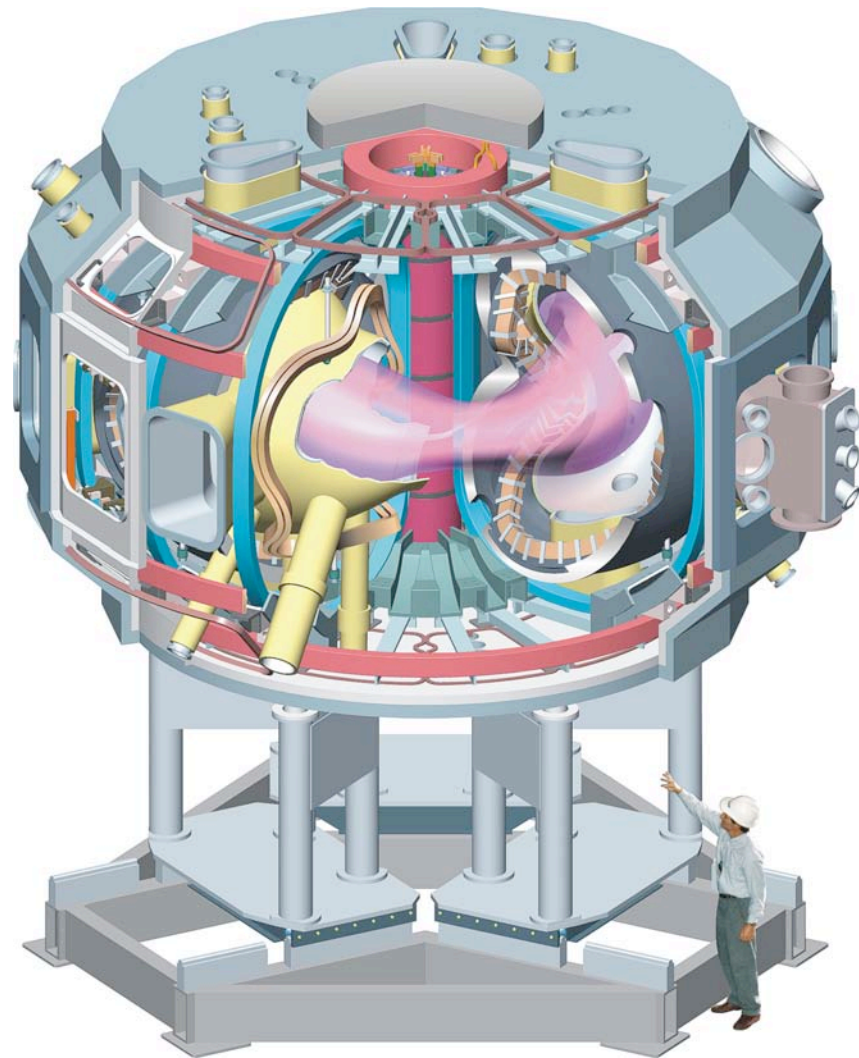
Vessel bakeable to 350 C.

Plasma Heating planned

NBI: 6 MW (tangential)

ICH: 6 MW (high-field launch)

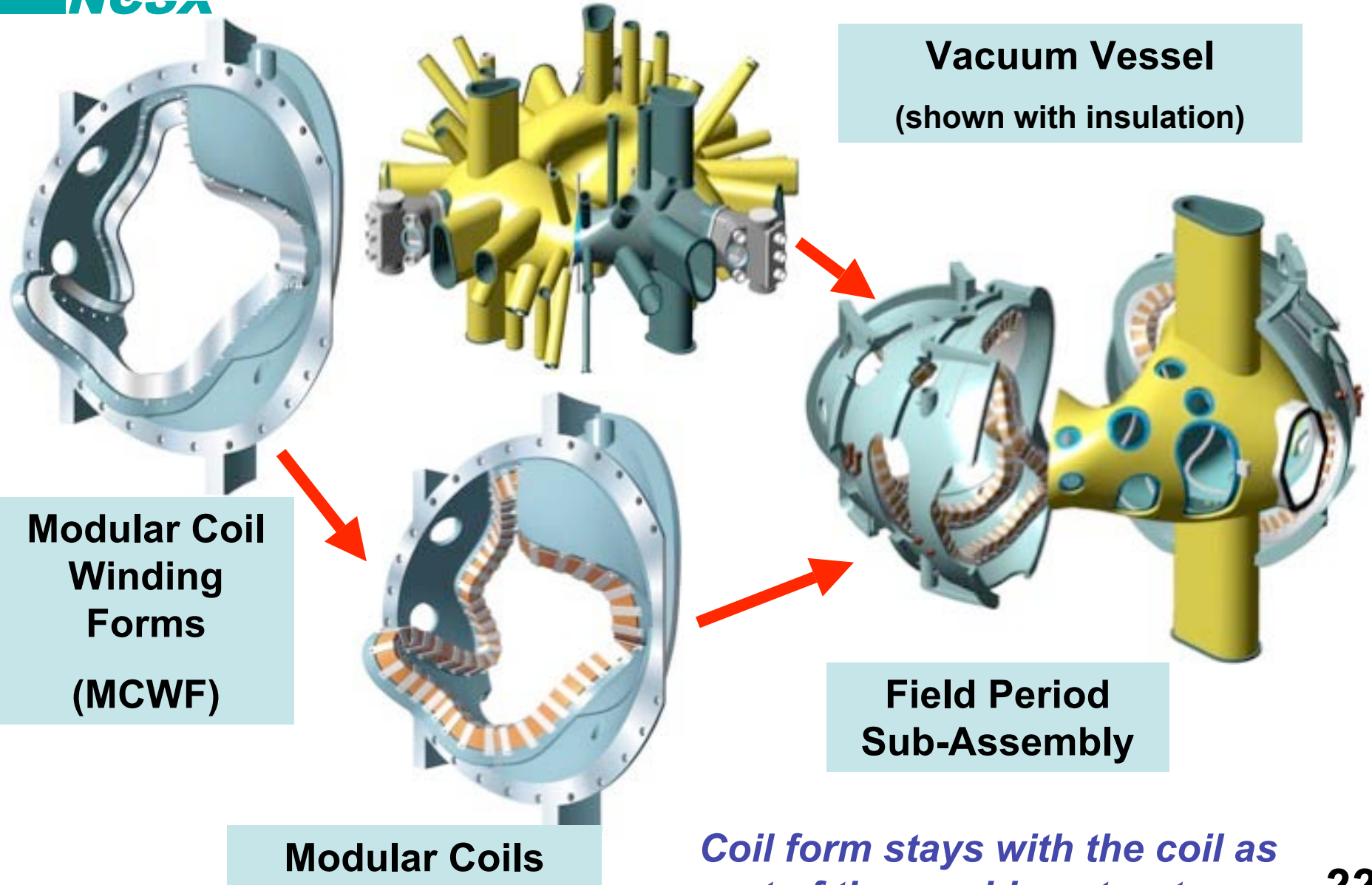
ECH: 3 MW



*coils cooled to cryogenic temperatures,
vacuum vessel at room temperature.* **20**

NCSX Engineering is Based on a Robust Concept

NCSX

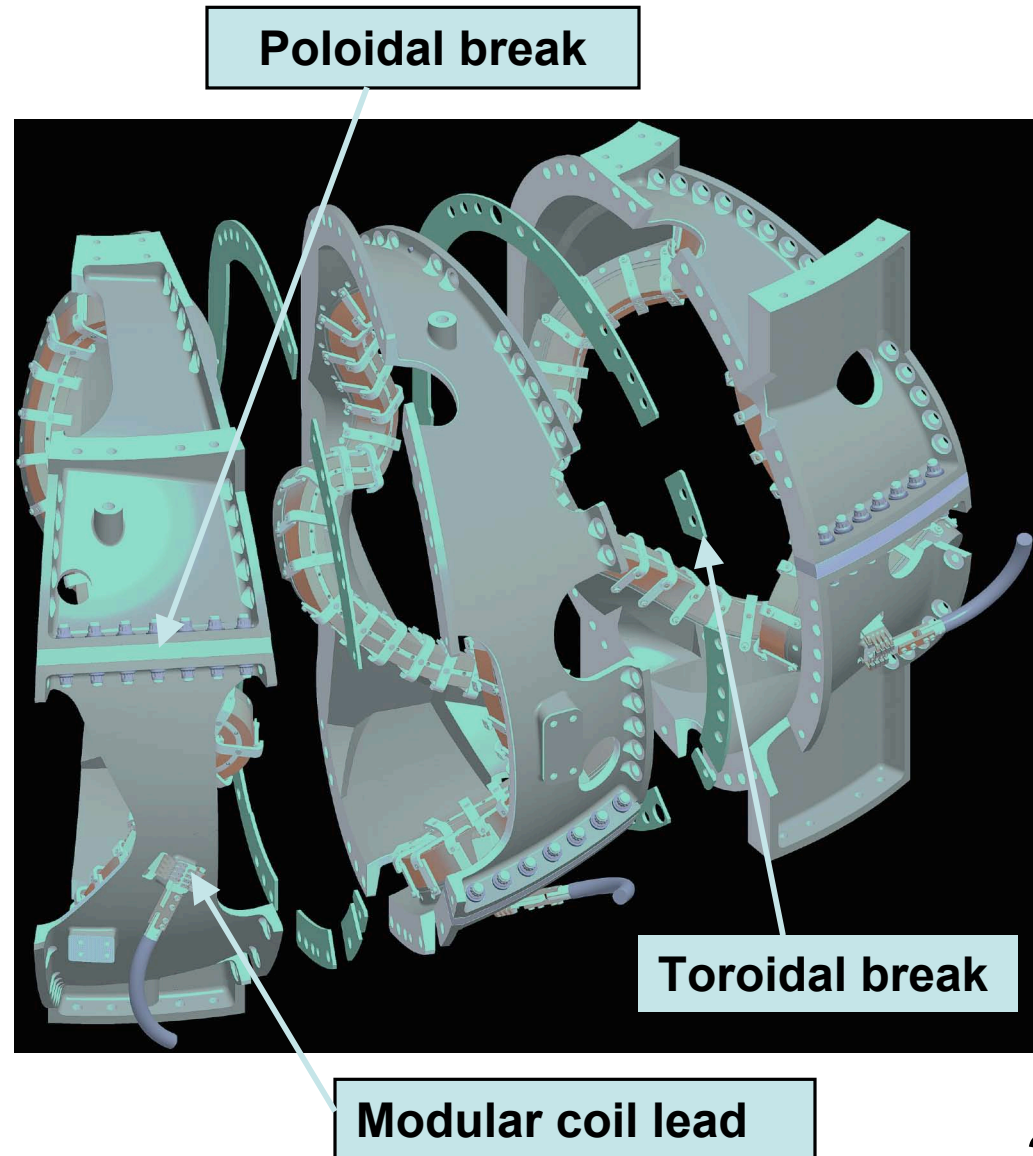
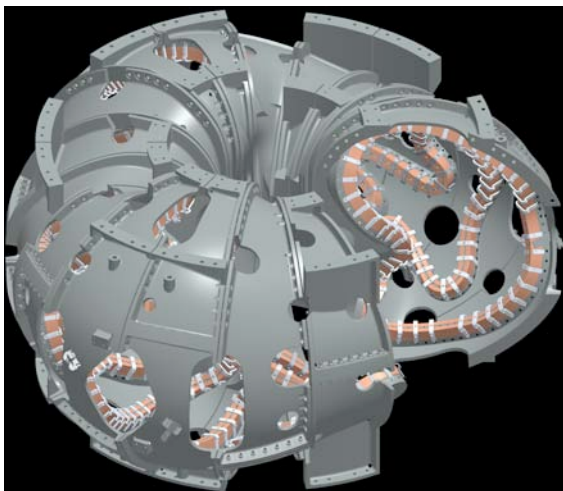


Coil form stays with the coil as part of the machine structure.

Winding Forms Make a Continuous Shell

NGSX

- Shell consists of individual modular coil winding forms that are bolted together
- Insulating breaks reduce eddy currents.
- *Shell concept also attractive for reactors. –ARIES-CS.*



Vacuum Vessel Has Good Access



- Interior space maximized for SOL and divertor flexibility, consistent with assembly of coils over vessel.
- Port configuration maximizes diagnostic access.
- Vacuum vessel bakeable to 350C means future PFCs can be simpler, more reliable, take up less space.

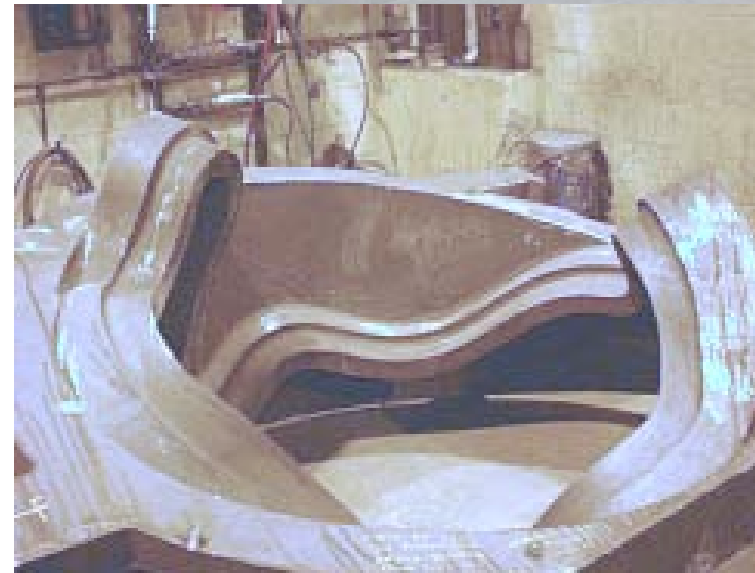


- **Shell material** - - - - -Inconel 625
- **Thickness** - - - - - 0.375 inch
- **Time constant** - - - - -5.3 ms

MCWF and VV Manufacture Have Started!



- Industry teams developed solutions to fabrication challenges through R&D.
 - Geometries, tolerances, materials.
- Fabricated prototypes.
 - Using product data close to final design specs.
 - Gained experience, reduced production risks.
- **We have placed contracts for the production components**
 - VV: Major Tool & Machine, Inc., Indianapolis, IN
 - MCWF: Energy Industries of Ohio, Inc., Cleveland, OH, with:
 - C.A. Lawton Co. (pattern)
 - Metaltek International (casting)
 - Major Tool & Machine (machining)



Preparing to Wind the Modular Coils

NCSX

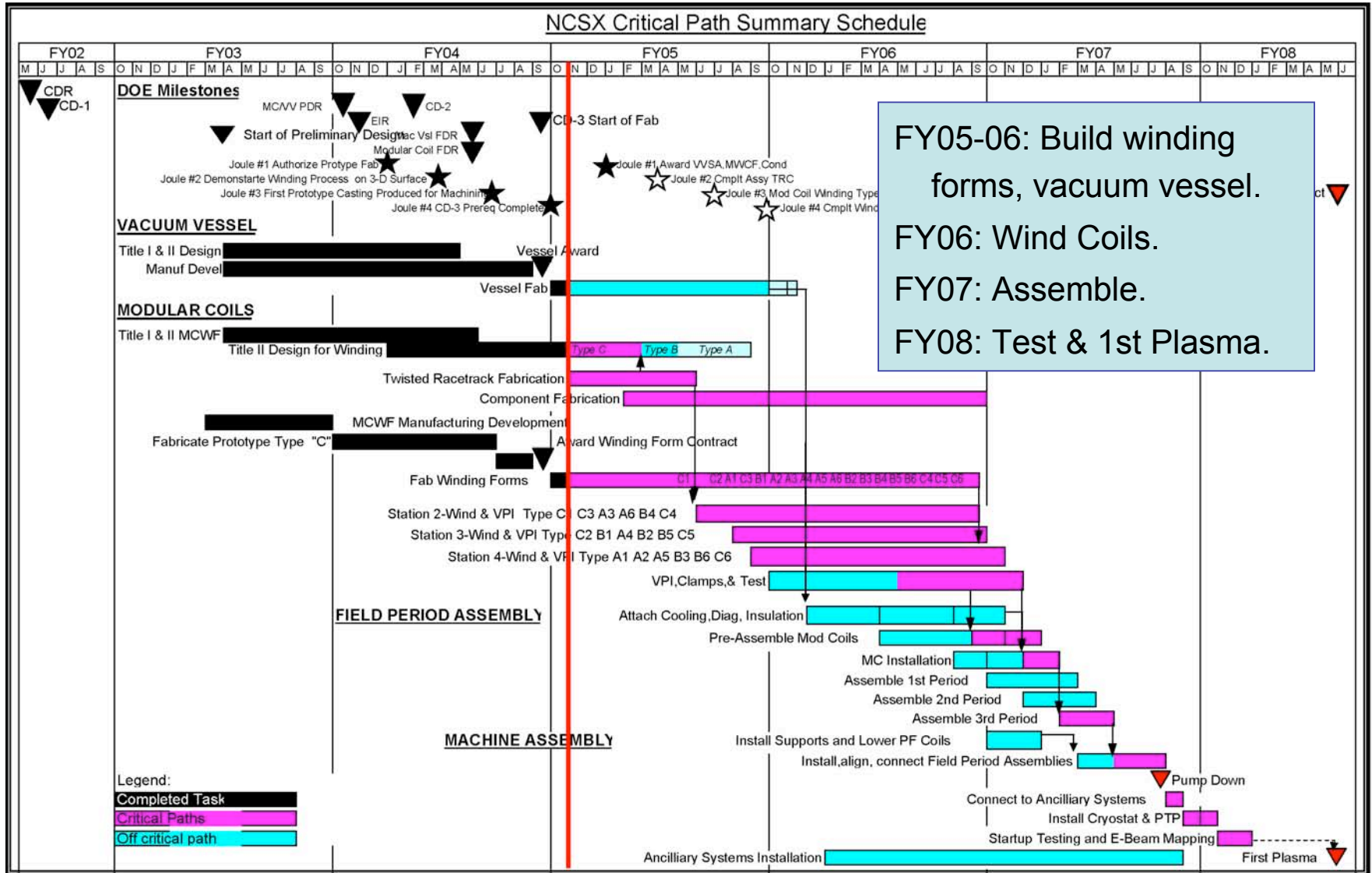
- Manufacturing facility is operating.
- Process steps have been developed by R&D.
 - Conductor placement on 3D surface.
 - Conductor deformation during winding.
 - Tooling & fixture optimization.
 - Epoxy impregnation.
- Twisted racetrack will provide integrated demonstration.
- All coils will be cryogenically and electrically tested.

Modular Coil Manufacturing Facility



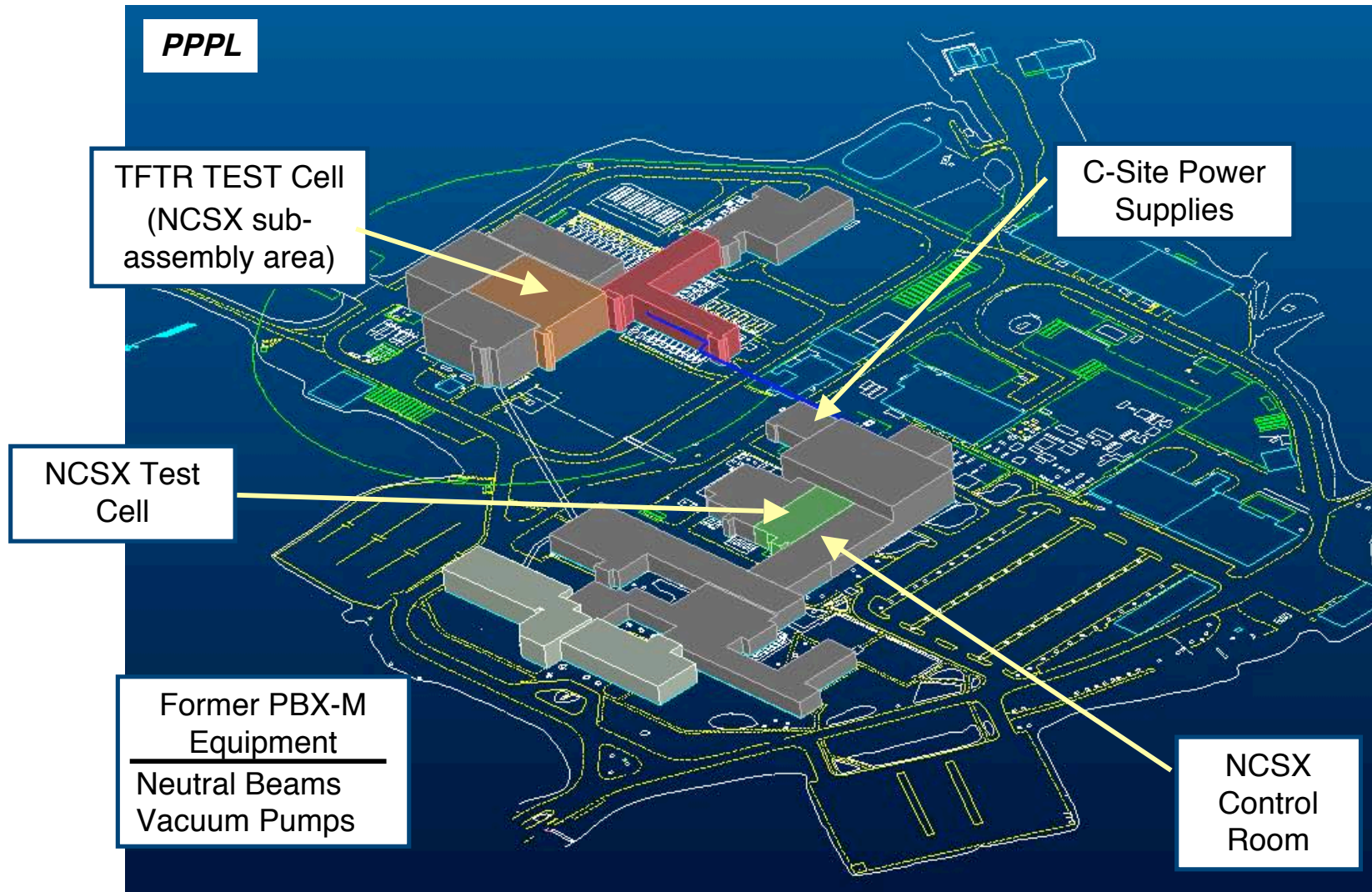
**Twisted Racetrack
Being Wound**

NCSX is on Schedule



NCSX Will Use Existing Fusion Program Infrastructure to Reduce Costs

NCSX



Research Preparations Are Under Way



- Developing requirements for magnetic sensors, first wall design, diagnostics.
- Developing analytical tools.
- Collaborating on stellarator experiments.
- Planning the experimental program.
- Program Advisory Committee meets annually.
- Research forum in late 2006.

NCSX research will be a national and international collaboration led by PPPL-ORNL partnership.

Summary



- Stellarators advance fusion science and provide solutions to magnetic fusion challenges:
 - Steady state, high-beta operation.
 - Understanding of 3D plasma physics for all MFE
- The NCSX is designed around a low- R/a , high-beta, quasi-axisymmetric stellarator plasma and a flexible coil set.
- Construction of major components has started.
- First Plasma in May, 2008.
- Research to be a national / international collaboration.