

RECENT RESULTS FROM DIII-D PERTAINING TO FIRE

by
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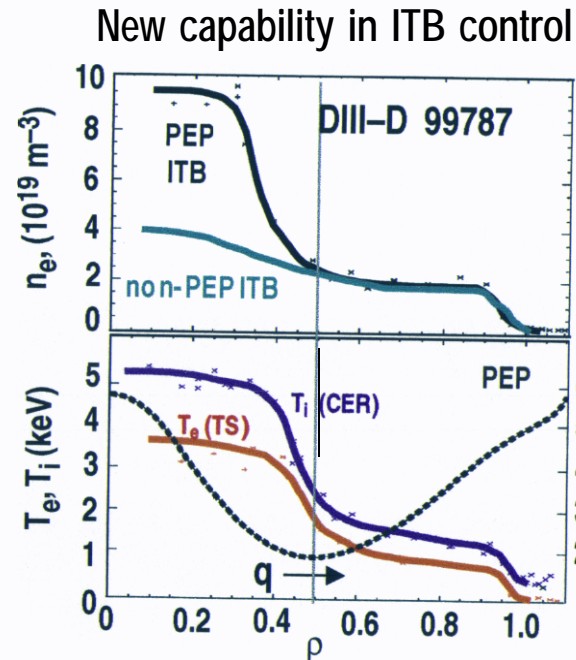
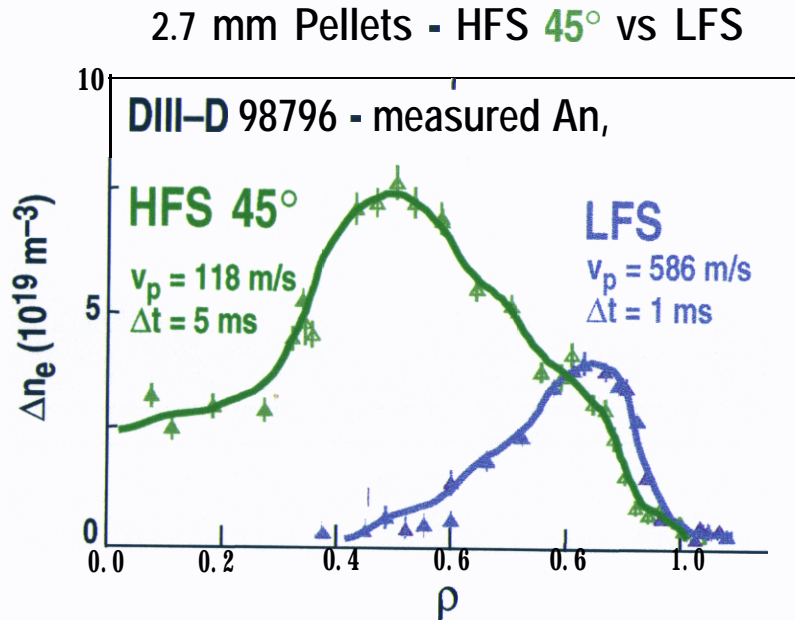


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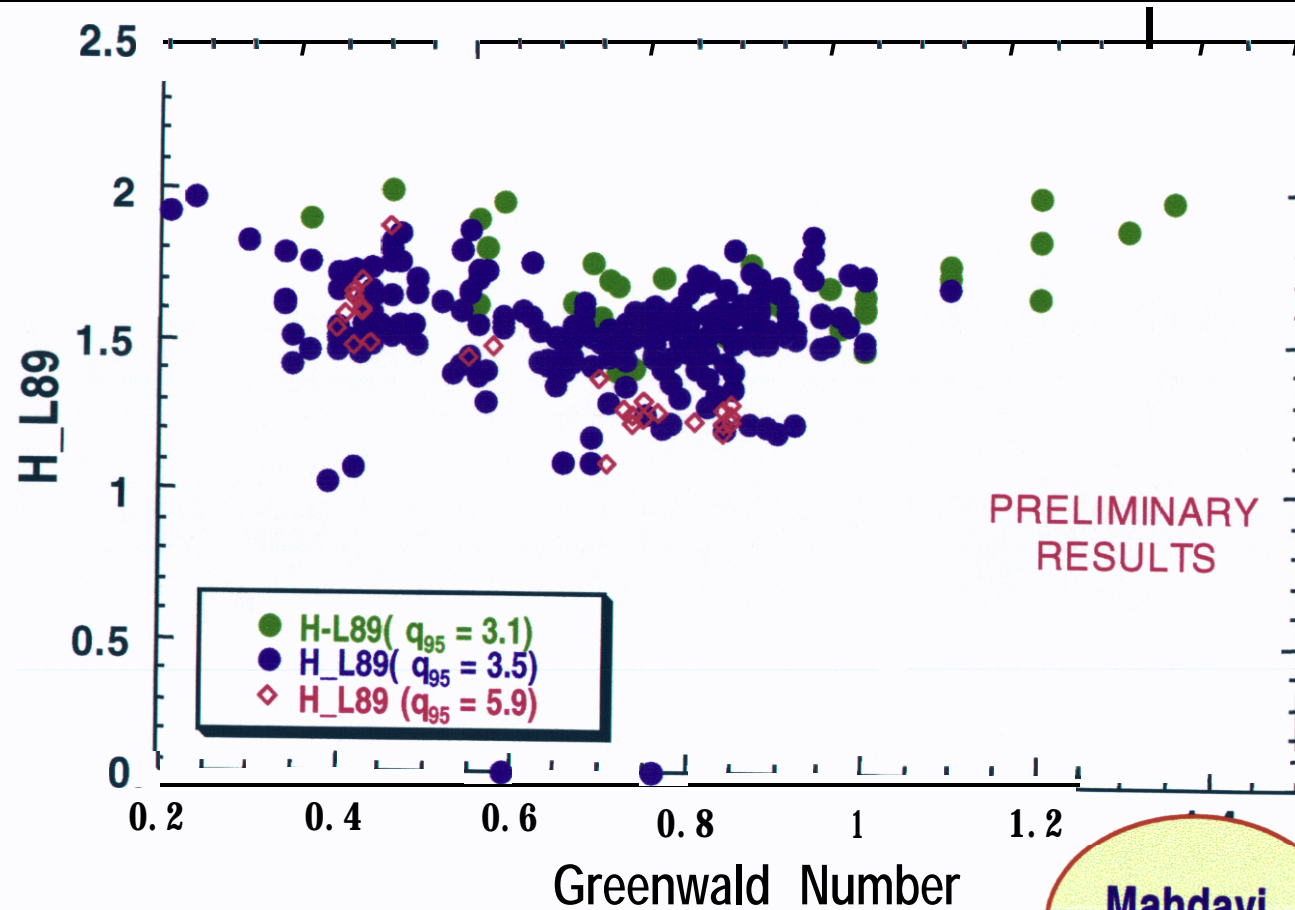
1. High density operation
2. Advanced tokamak regime
3. Beta limits
4. Normalized current
5. Summary

HIGH FIELD SIDE PELLET INJECTION ALLOWS EVALUATION OF INTERNAL TRANSPORT BARRIERS WITH $T_e \sim T_i$



- HFS pellet injection yields deeper particle deposition than LFS injection, consistent with theory
- Future work on ITB control and H-mode control with pellet

CONFINEMENT DOES NOT DEGRADE AT HIGH DENSITY IN LOW q_{95} PUMPED DISCHARGES



Summary and Conclusions of High Density Experiments

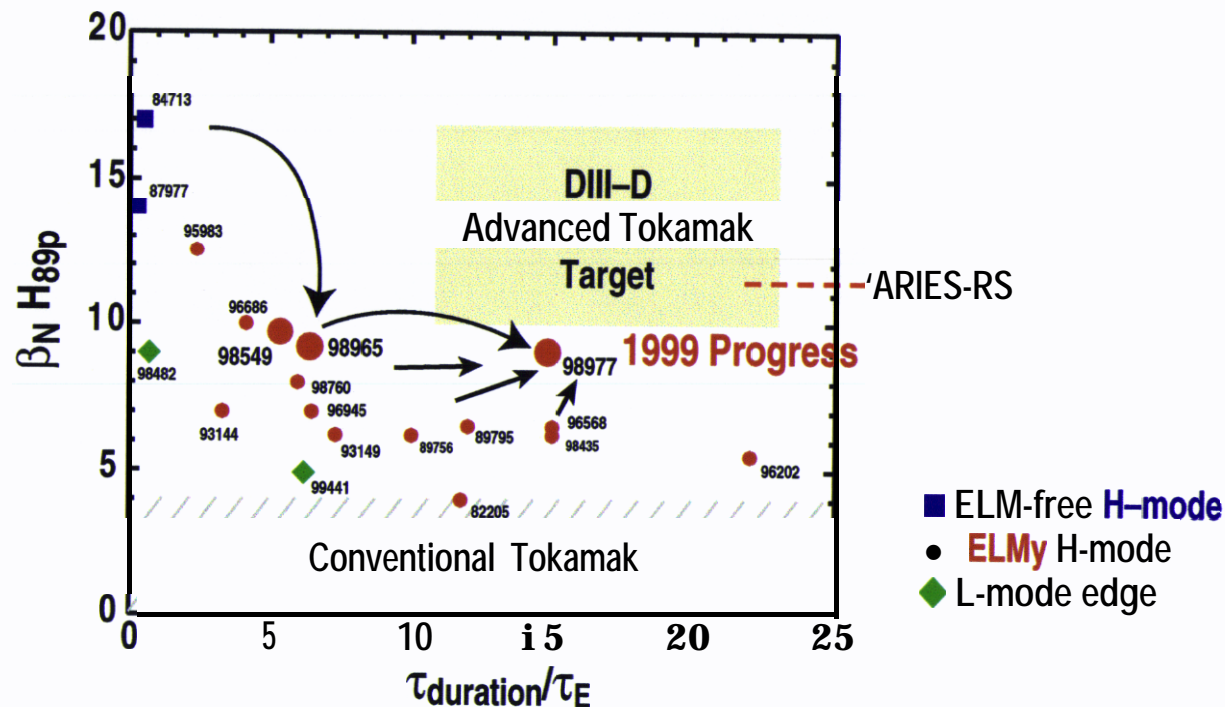
- Densities up to $1.4n_{GW}$, and $H89p \sim 1.9$ are obtained with **divertor** pumping and gas fueling alone
 - ⇒ Pedestal densities up to $0.9 n_{GW}$ observed
 - ⇒ Pumping seems necessary for maintaining high confinement
 - ⇒ Best results obtained at low q and low power
- Good confinement is correlated with good fueling efficiency

Motivation for Improved Performance

- Magnetic fusion reactor
maintain high fusion power density ($\propto \beta^2 B^4$)
- Steady-state $\Rightarrow f_{bs} \approx 1$ (in tokamak)
increasing the bootstrap fraction means increasing q
 \Rightarrow high q ($\propto f_{bs}^{1/2}$)
- Stability must be improved
 \Rightarrow increase β_N ($\propto q$)
- Must exceed ignition condition & maintain power balance during burn
(maintain $P_{fusion}/P_{loss} \propto \beta\tau$)
 \Rightarrow increase H ($\propto q$)
- For example: if a tokamak reactor plasma has $q \approx 3$, $\beta_N H_{89p} \approx 5$, and $f_{bs} \approx 40\%$,
to reach $f_{bs} \approx 100\%$ at the same β would require $\beta_N H_{89p} \approx 12.5$ at $q \approx 4.7$.

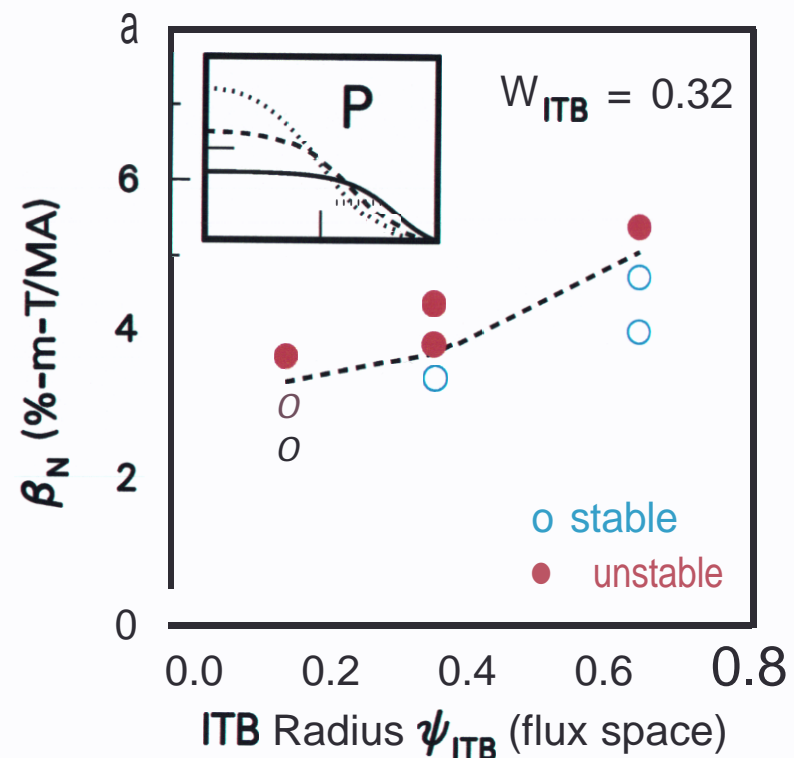
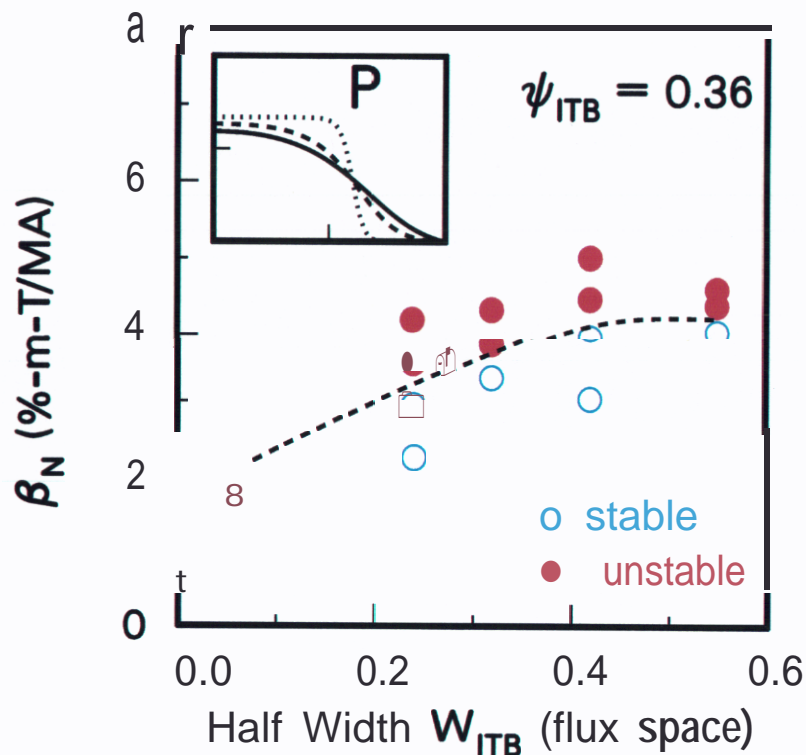
DIII-D Goal and Progress in 1999

A principal near-term goal of the present **DIII-D** research program is a stationary plasma with $\beta_N H_{89p} \geq 10$, with no inductive current, a relaxed loop voltage profile, and $> 50\%$ bootstrap current.

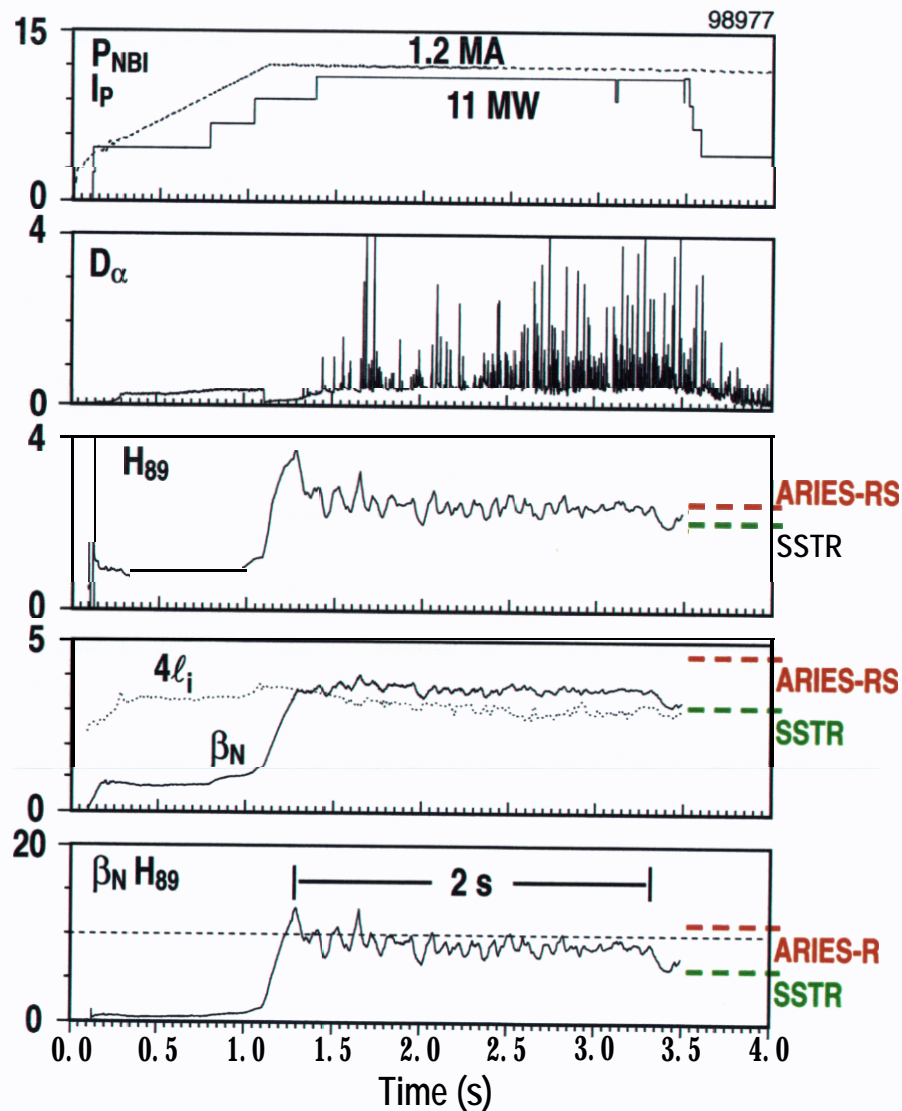


STABILITY LIMIT IMPROVES WITH INTERNAL TRANSPORT BARRIER WIDTH AND RADIUS

- Fixed shape, DND, $q_{95} \square 5.1$, $q_0 \square 3.2$, $q_{\min} \square 2.2$ based on a **DIII-D** discharge
- Hyperbolic tangent pressure representation
- Ideal $n = 1$, wall at **1.5a**



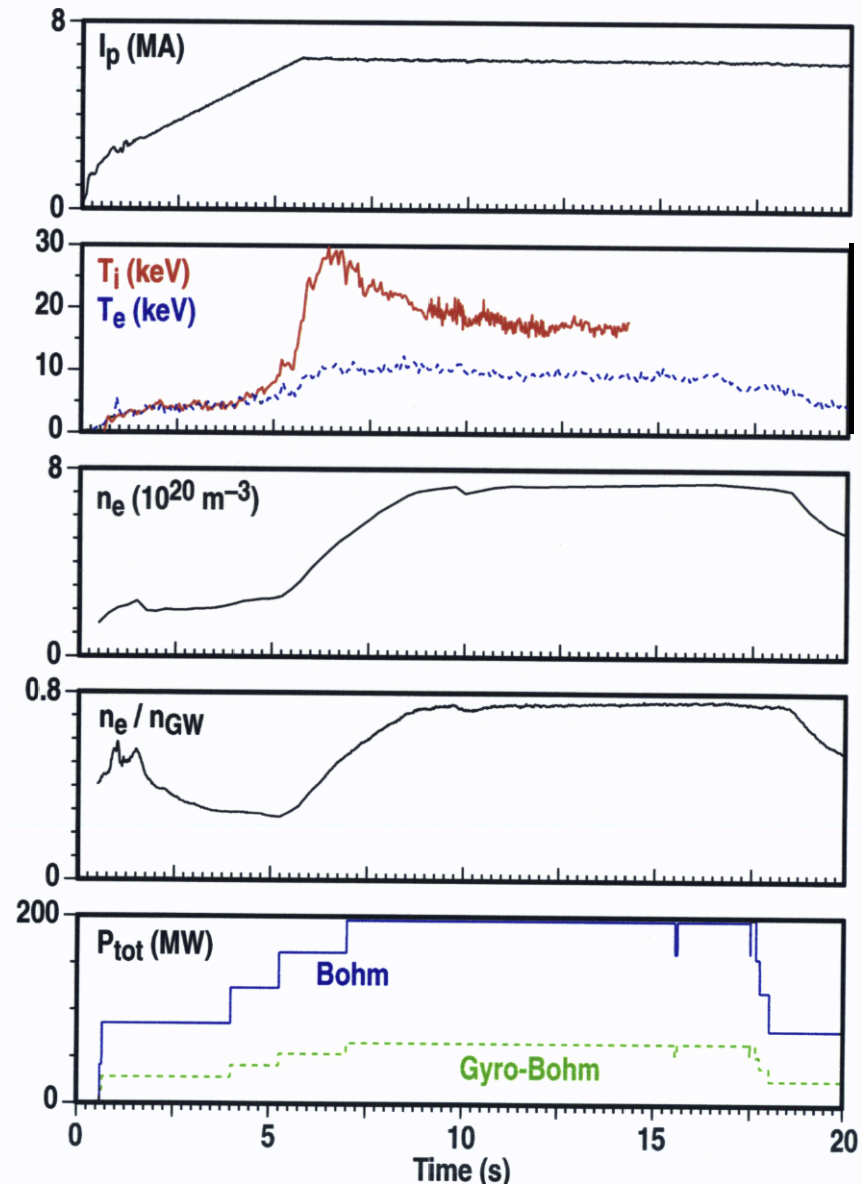
$$\beta_N H_{89p} \geq 9 \text{ for 2 sec (16 } \tau_E \text{ \& } \sim 1 \tau_R)$$



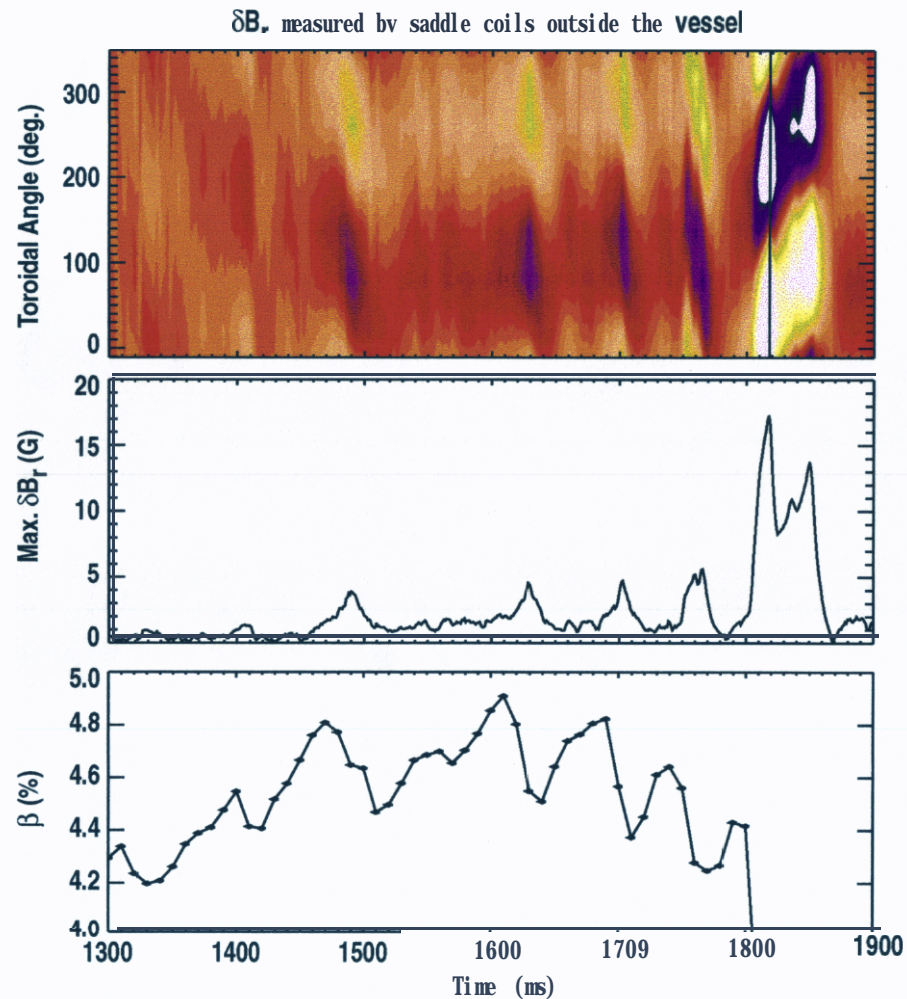
- Discharge preparation to produce hot core with hollow current profile
- Confinement meets reactor requirements
- β exceeds no-wall limit – no reduction when **ELMs** start
- Flat-top β limited by a combination of high frequency modes, **RWMs**, and **ELMs**
- Duration is many τ_E – comparable to current relaxation time

ADVANCED TOKAMAK DISCHARGE ON DIII-D SCALED TO FIRE AT FIXED β , ν AND B_p/B_T BUT SMALLER ρ_*

- FIRE has higher aspect ratio than DIII-D and thus will have lower safety factor and bootstrap fraction
- If current is reduced to increase bootstrap fraction, then density will exceed the density limit



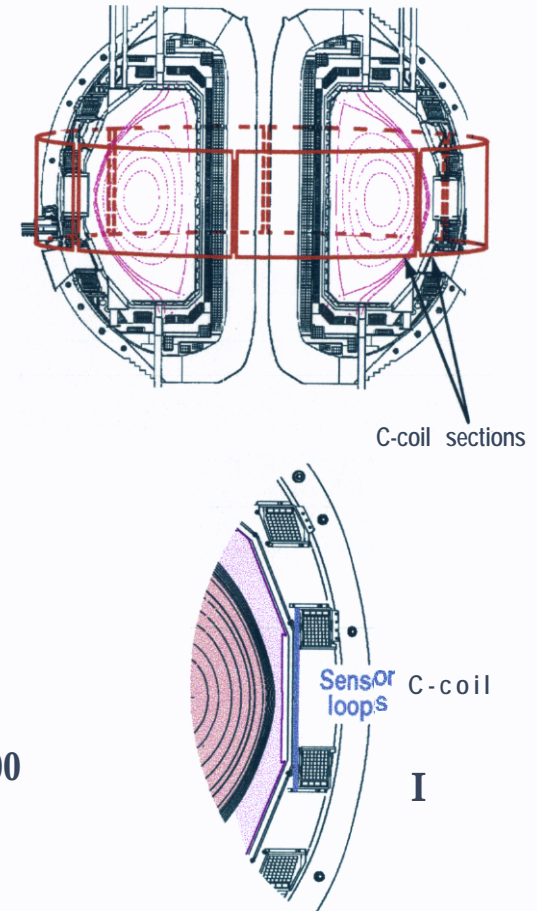
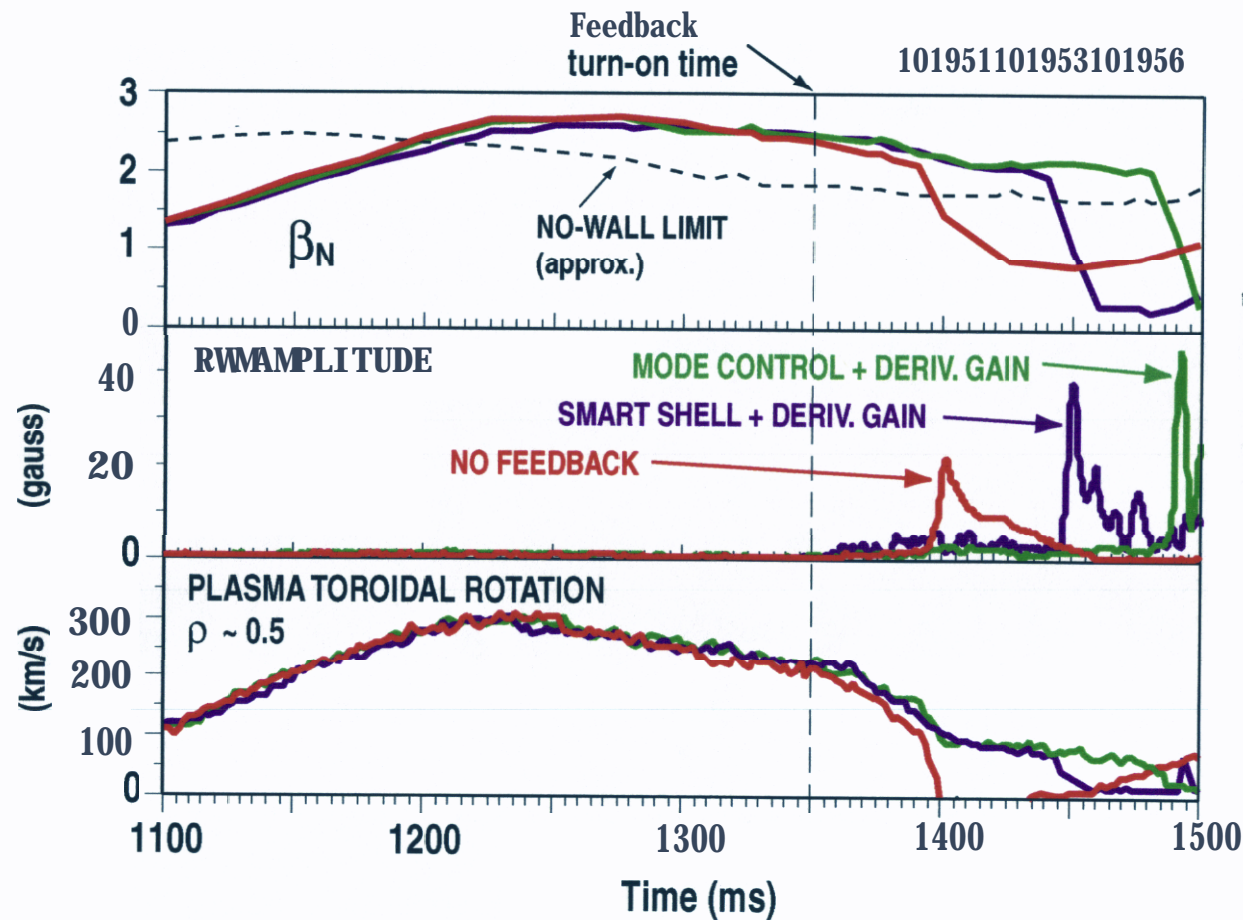
BETA IS LIMITED IN MAGNITUDE AND DURATION BY RESISTIVE WALL MODES



- Limiting modes have the characteristics of resistive wall modes: $\gamma \sim 1/\tau_w$
- Rotational stabilization is difficult to maintain
 - Theoretical understanding of drag mechanism needed
- ⇒ Active feedback stabilization is required
- RWM stabilization is a key issue for RFP, ST, and spheromak (FESAC goal #2)

ACTIVE FEEDBACK STABILIZATION EXTENDS HIGH β DURATION

(March 2000)



PLASMA CURRENT IS WHAT MAKES TOKAMAKS GREAT

- Normalized current $\frac{I}{aB_T} = \frac{B_p}{B_T} \left(\frac{2\pi}{\mu_0} \right)$ is relevant quantity
- Beta increases with normalized current regardless of beta limit

$$\beta = \left(\frac{\beta_N}{100} \right) \left(\frac{I}{aB_T} \right)$$

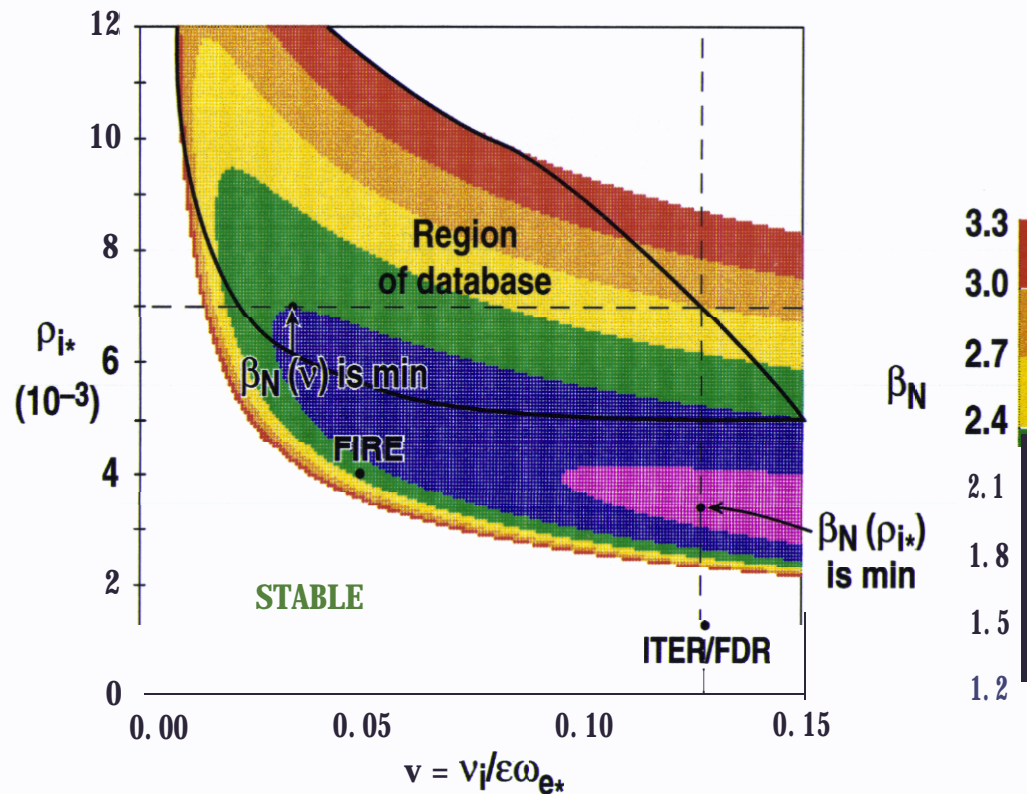
- Confinement increases with normalized current

$$B_T \tau_{th} \propto \rho_*^{-3.1} \beta^0 v^{-0.4} \left(\frac{I}{aB_T} \right)^{1.4} \text{ for DIII-D H-modes}$$

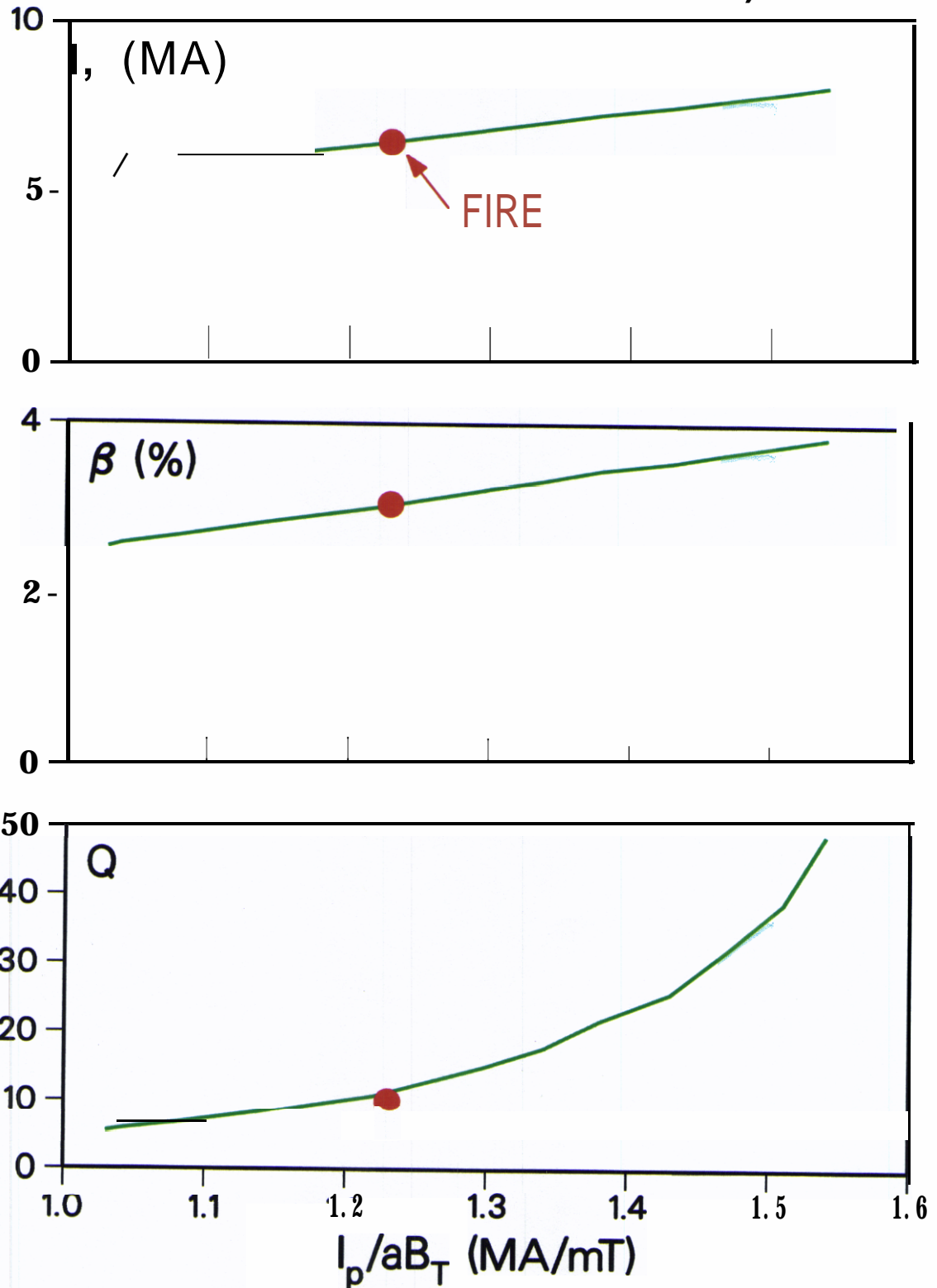
- Thus, the ignition criteria **Q** is a strongly increasing function of normalized current

NEOCLASSICAL TEARING MODE BETA LIMIT

- Model for onset β_N fitted to database for $m/n \propto 3/2$ NTM induced by **sawteeth**
 - ★ DIII-D, JET, and ASDEX-Upgrade (**ELMy** H, $q_{95} \gtrsim 3$, LSND)
 - ★ Seed island decreases faster with ρ_{i*} than threshold
 - Stabilizes at very low ρ_{i*} , high S , i.e. $w_{\text{seed}}/w_{\text{thresh}} \propto 1$
 - ★ FIRE is predicted to be unstable at $\beta_N \approx 2.1\text{-}2.4$



THE IGNITION CRITERIA Q RAPIDLY INCREASES
WITH NORMALIZED CURRENT (R , B_T AND β_N
FIXED TO FIRE VALUES)



SUMMARY

- Good confinement at high density is correlated with good fueling efficiency
 - Best results obtained at low beta
- Long duration AT regime ($\beta_N H = 9$) has broad pressure profile and beta only $\approx 10\%$ above ideal no wall limit
 - Obtained in hot-ion mode with rapid rotation
- Neoclassical tearing mode predicted to be unstable for sawtoothed plasmas in FIRE around $\beta_N = 2.1\text{--}2.4$