

Overview of the last HT-7 experiments

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General Atomic, San Diego, USA

National Institute for Fusion Sciences, Toki, Japan

Advanced Fusion Research Center, Kyushu University, Japan

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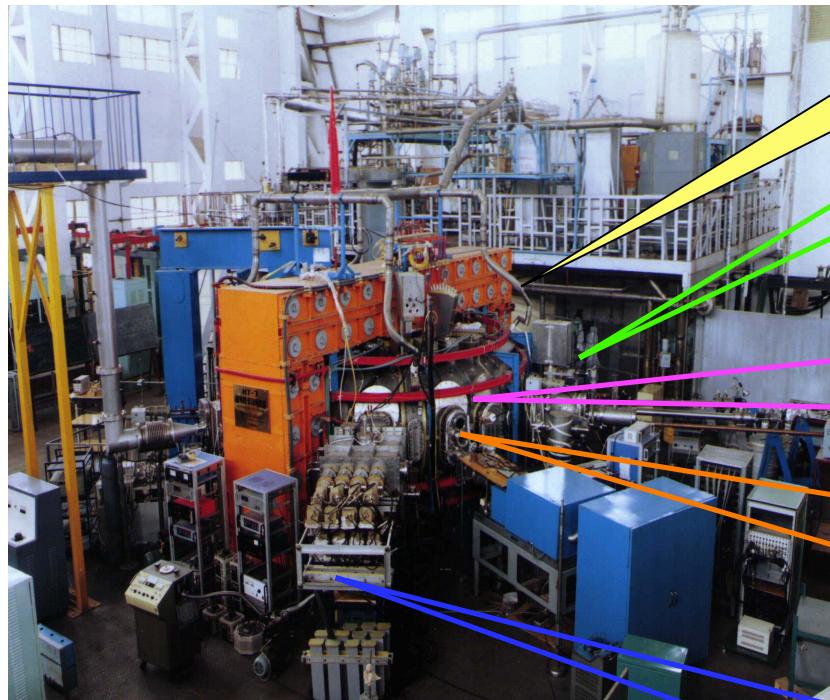
FEC-2004 OV/5-1Rb

Outline

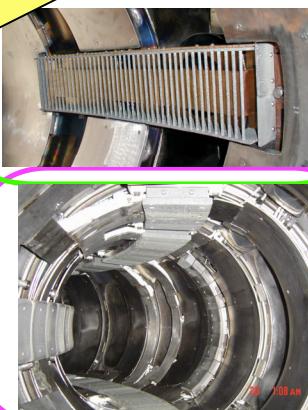
- HT-7 mission and R&D
- Confinement improved scenarios
- High performance under steady state
- Long pulse discharges
- Summary



Main Mission: Steady-state high performance operation and related physics and technologies



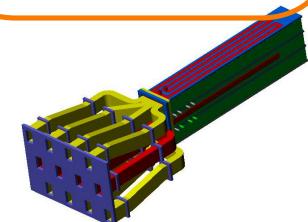
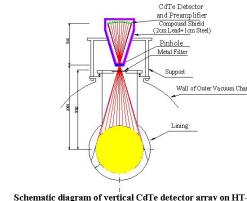
SS real-time PF
PS&control



RF: 18-30MHz
350 kW CW
IBW, FW antenna

Graphite (P, T)
Water cooled

>30 diagnostics
Steady-state



f: 2.45 GHz
1.2 MW CW
Multijunction

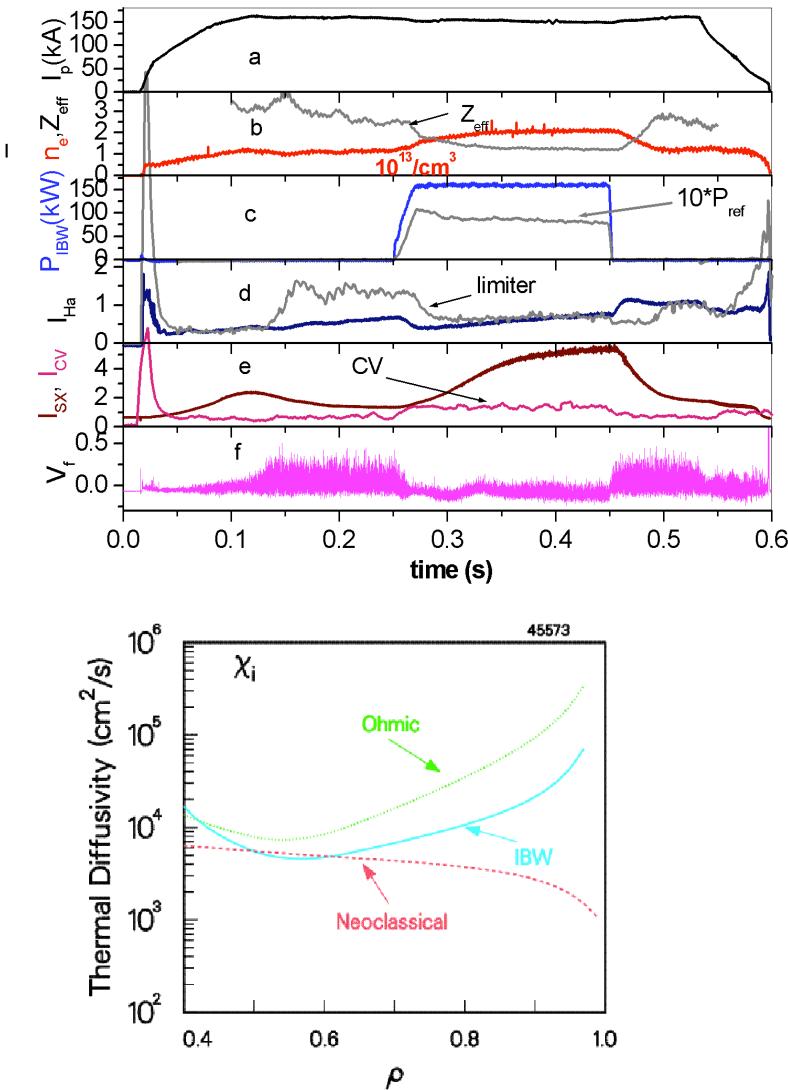
$$R = 1.22\text{m}, \quad a = 0.27\text{m},$$

$$I_p = 100\text{--}250 \text{ kA}, \quad B_T = 1\text{--}2.5 \text{ T}$$



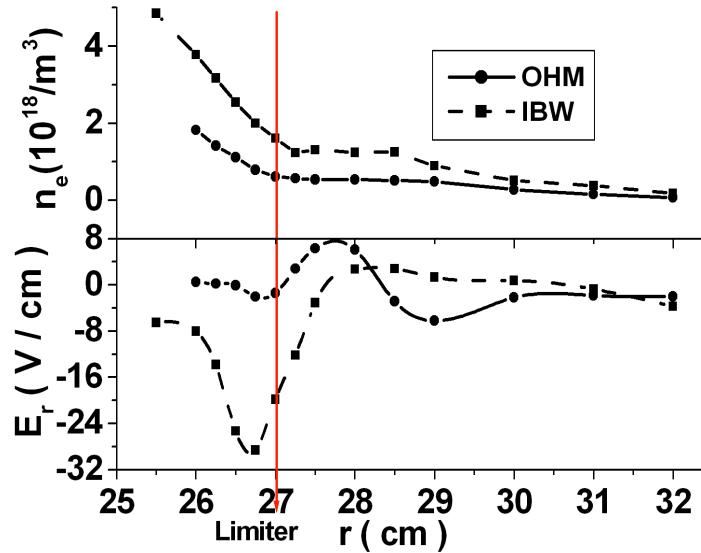
H-mode by IBW at 30 MHz

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IBW enhanced the edge E_r shear,
 $\omega_{E \times B} > \Delta\omega_t \sim ck_\theta T_e/eBL_n$ (for
drift-wave-like turbulence)

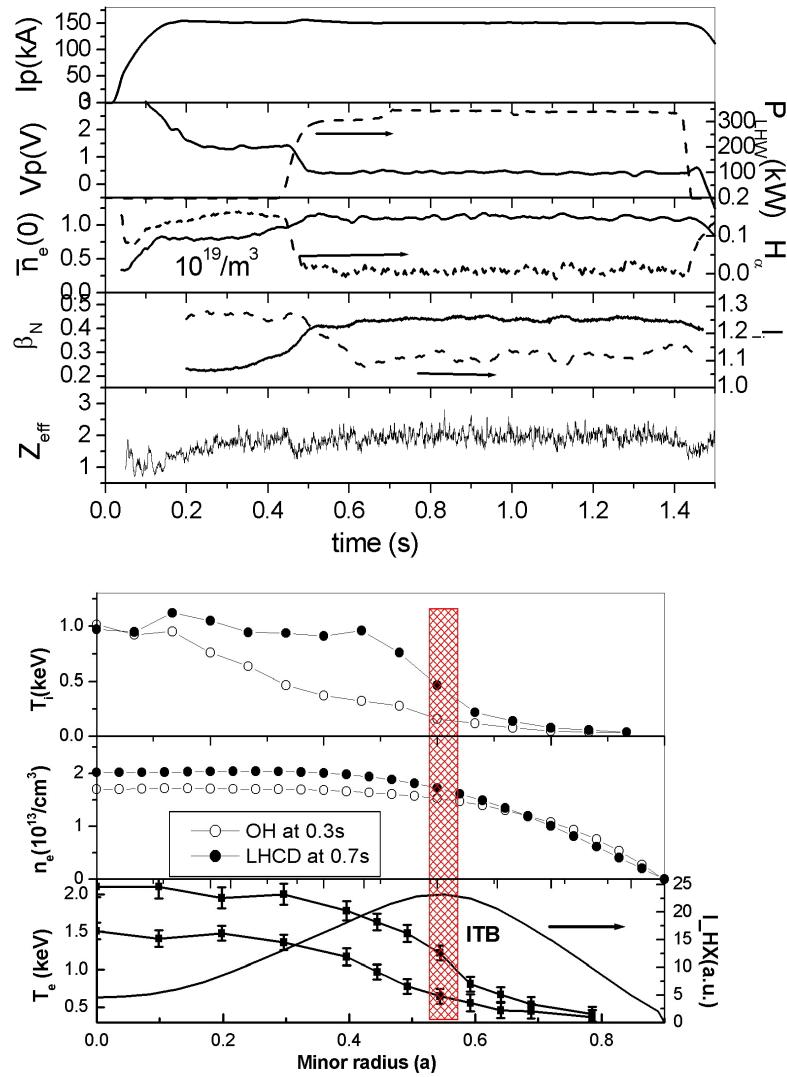
Suppression of edge turbulence
leading to improved confinement
Reduced transport in $\rho > 0.5$



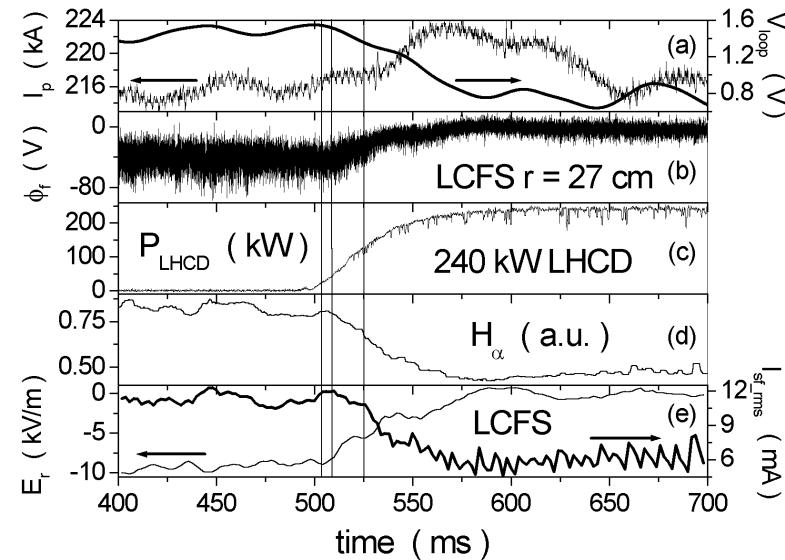


H-mode by off-axis LHCD

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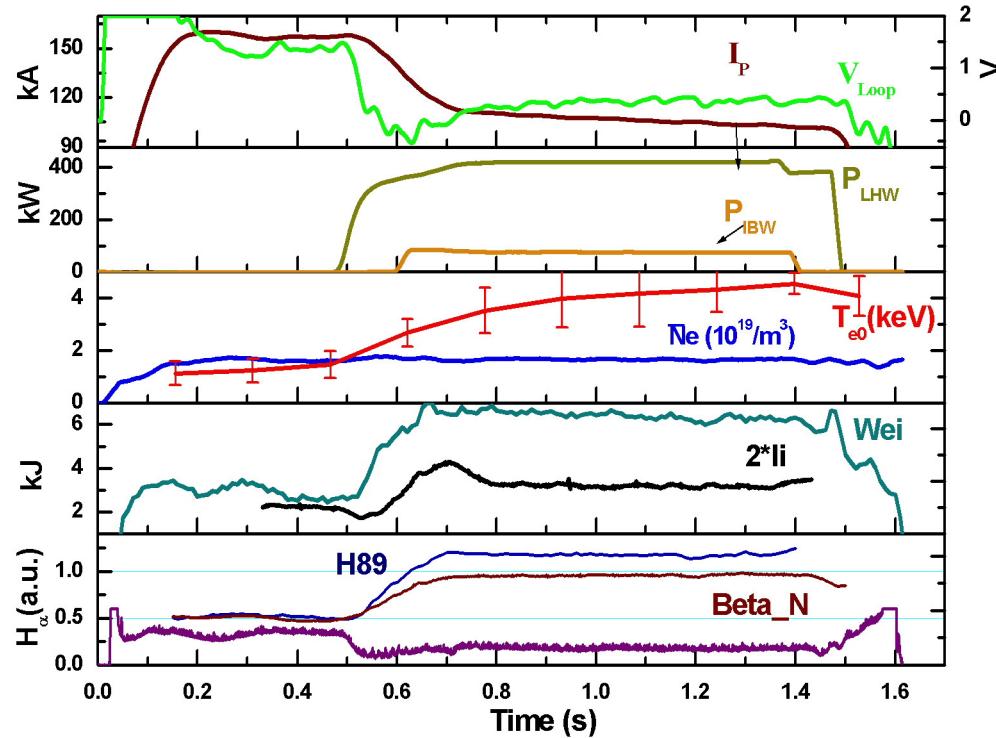
- Current density profile control by changing n_{\parallel}^{LHW}
- $H_{89} \leq 2$, with potential steady-state capability, but low β
- Enhanced edge E_r shear led to suppression of edge turbulence
- Improved confinement



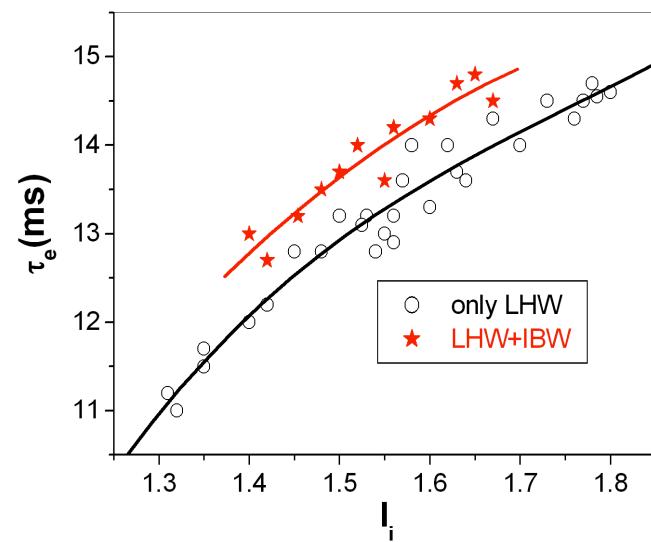
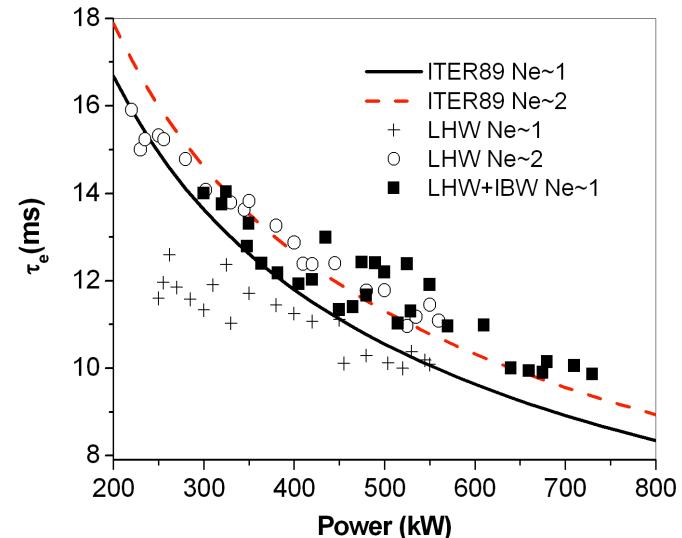


High li mode

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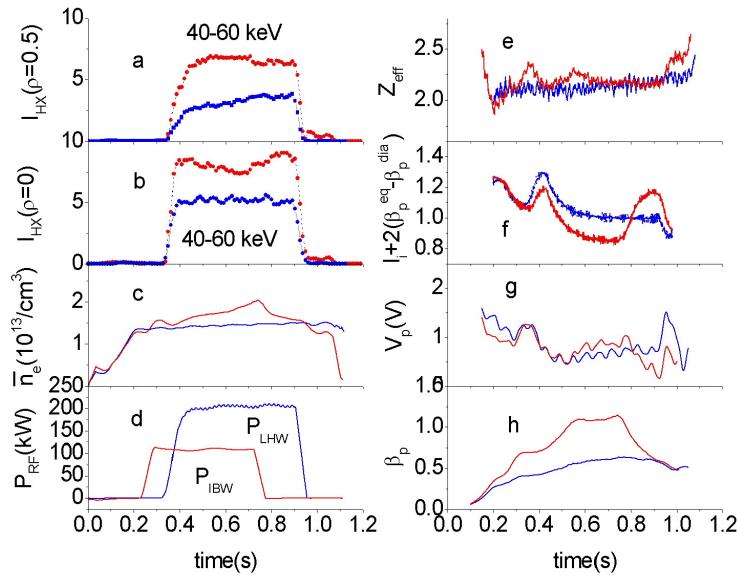
- I_p ramp-down at rate of $-0.5 \sim 1.2$ MA/s
- Strong peaked $T_e(r)$, $T_{e\max} \sim 4.5$ keV
- Steady-state high $li \sim 1.3 \sim 1.8$ for several τ_{CR}
- Better confinement with LHW+IBW
- Better confinement for higher li



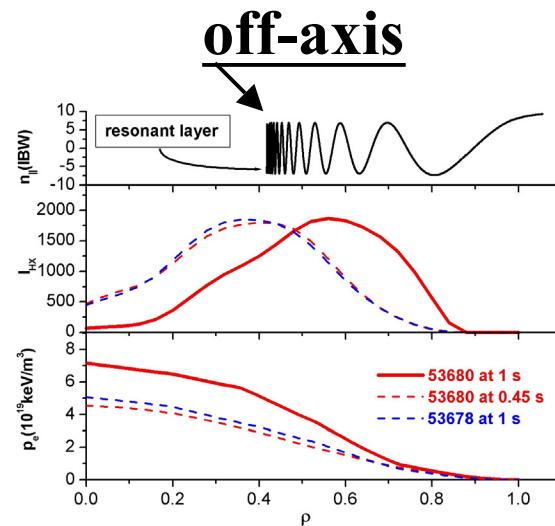
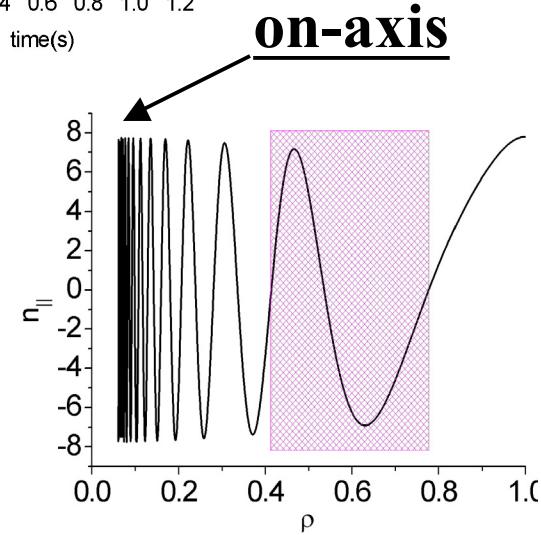
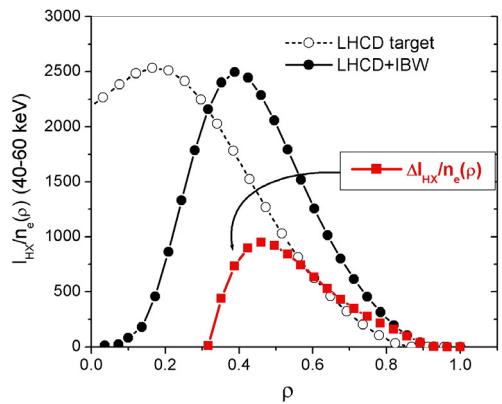


Local synergy of IBW and LHCD

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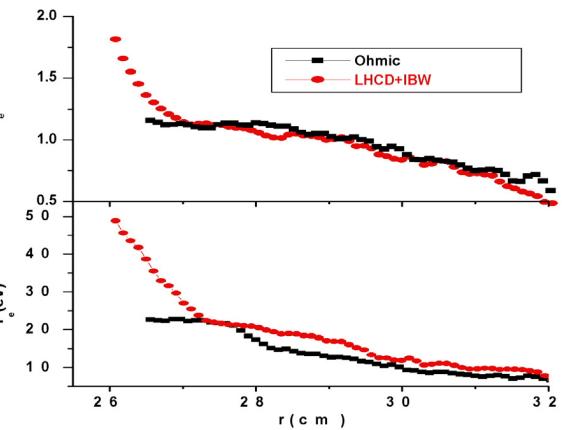
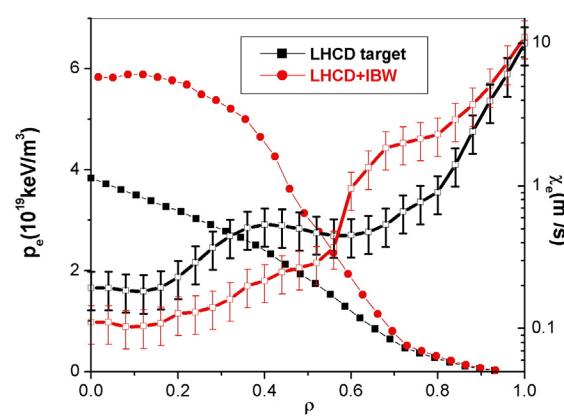
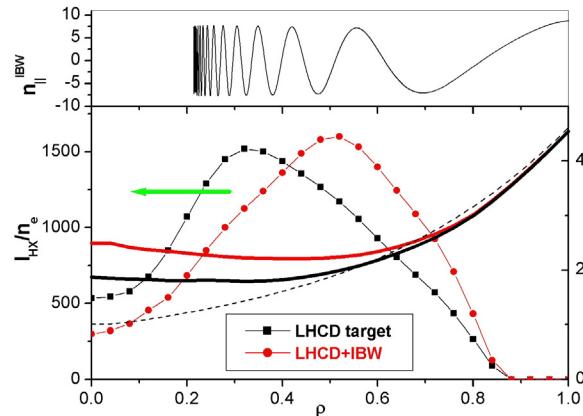
- $I_p = 150 \text{ kA}, N_{||} = 2.35$
- LHCD: near on-axis in the target
- IBW on-axis: localized HXR at $n_{||} (\text{max})$
- IBW off-axis: Enhanced HXR at $\sim \Omega_H$
- Broadened current profile
- Improved τ_E and τ_p & enhanced β
- potential SS capability



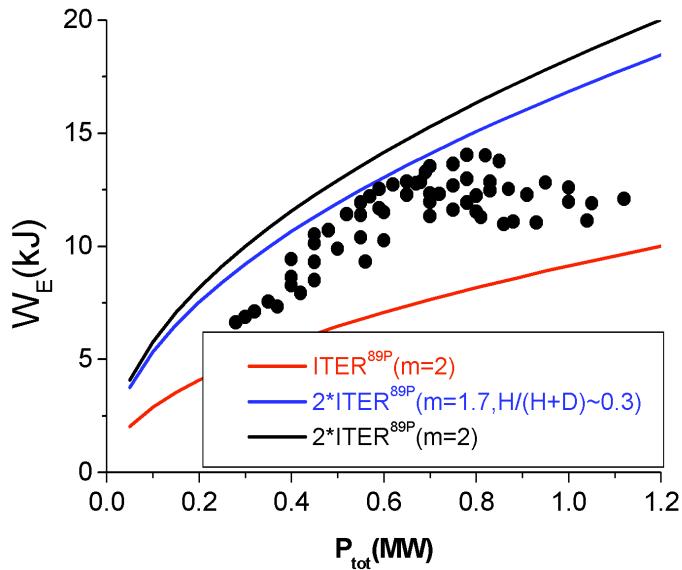


Features of IBW + LHCD synergy

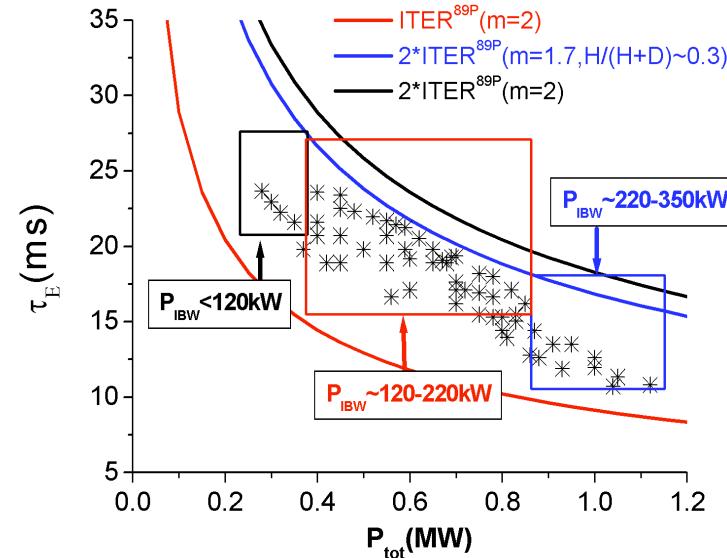
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Weak negative shear



ITB at minimal q



H-mode edge

Up to 1
MW good
confinement



Integration strategy

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Interaction of LHW and IBW can help localization of LHCD, create localized current channel and form weak positive or negative sheared q profile and ITB at well defined region by IBW.

To get high performance plasma under steady state condition, strategy was chosen to:

- Apply RF power at earlier phase of discharges.
- Avoid MHD instability via current density profile.
- Maximize plasma performance via pressure profile.
- Under RF boronized wall condition.

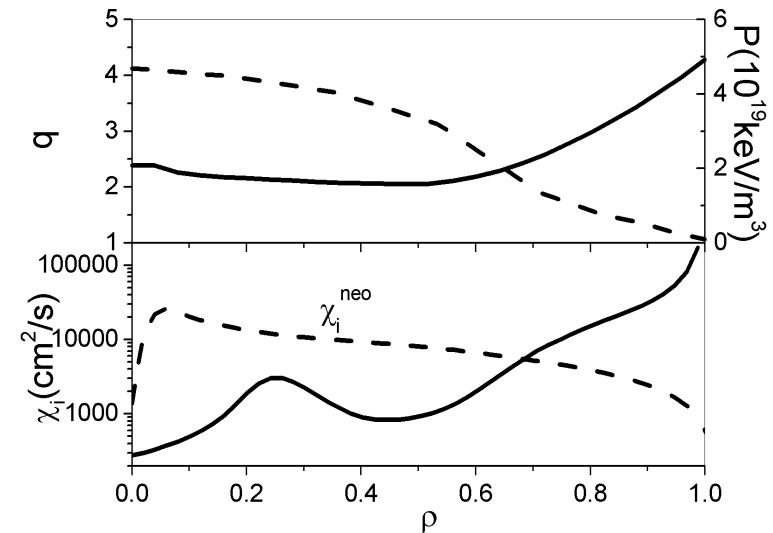
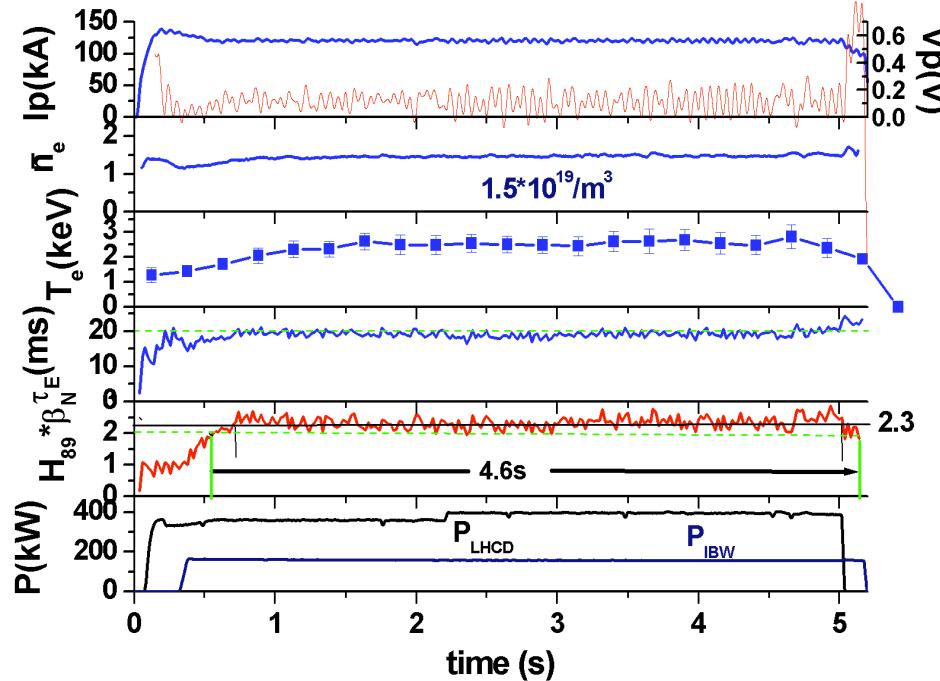
Compromise was made between performance and sustaining time.



Integrated high performance

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$H89^* \beta_N > 2.2$ for $\sim 220 \tau_E$ & $> 20 \tau_{CR}$ $f_{LHCD} + f_{BS} > 80\%$, $V_p < 0.10V$



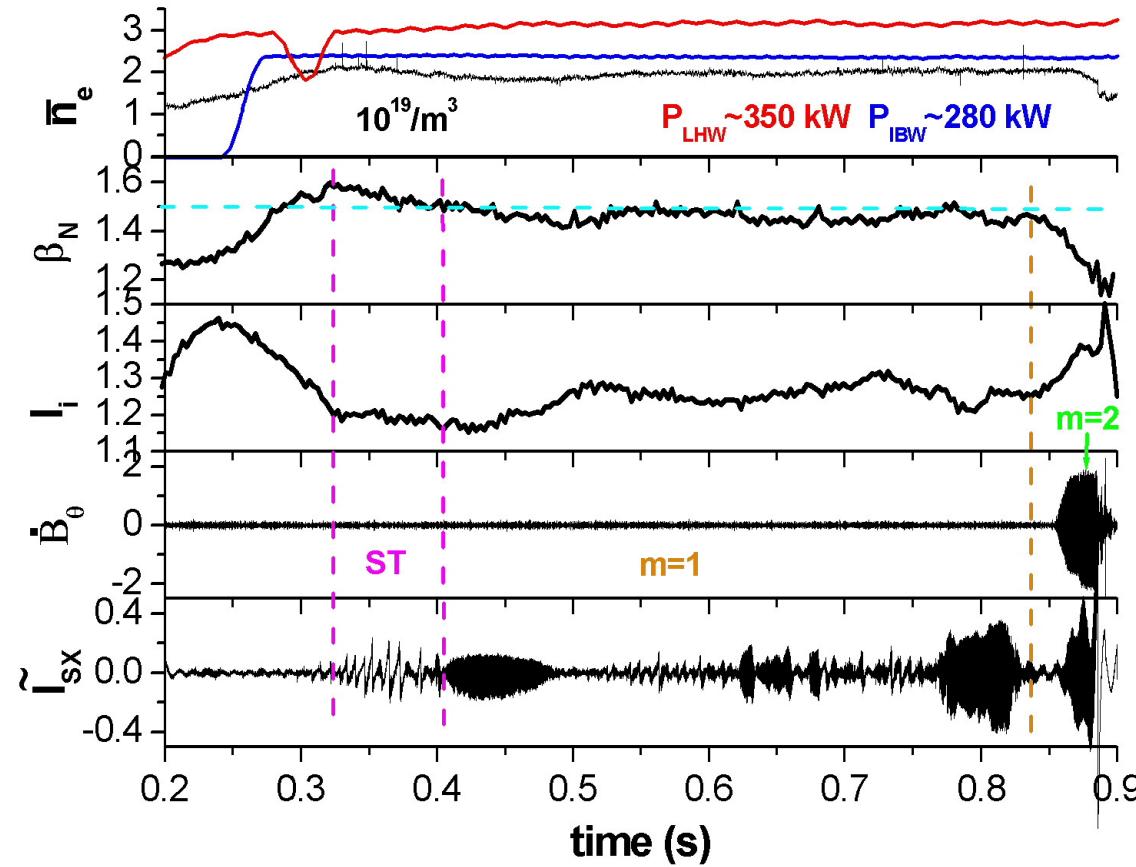
Stationary ITB was formed at footprint of minimal q and sustained during LHCD and IBW. $f_{LHCD} \sim 42\%$, $f_{BS} \sim 39\%$



Limitation of high performance

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$H_{89}^* \beta_N \sim 2.8$ MHD instabilities limit higher β_N

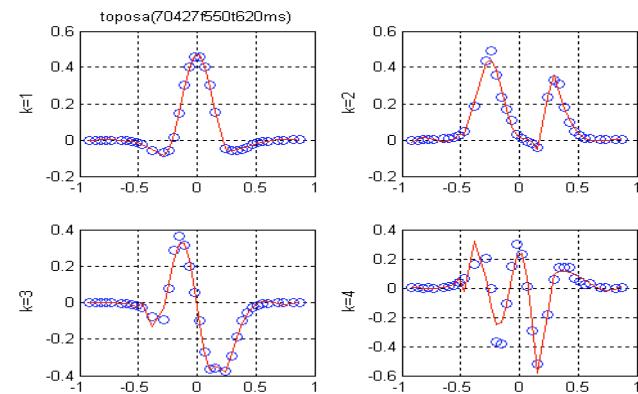
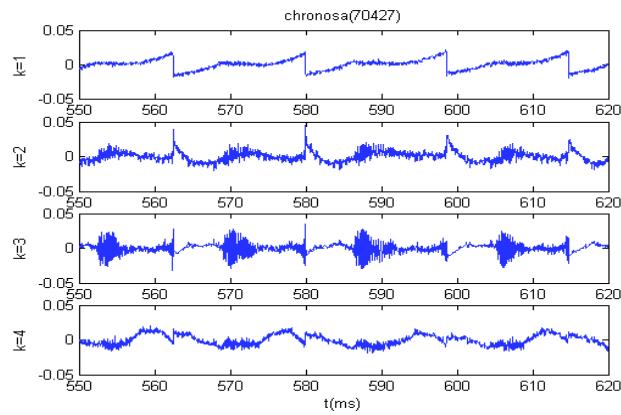
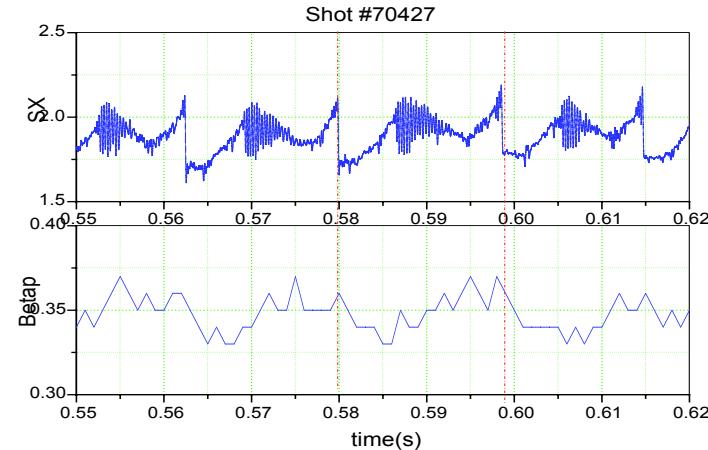
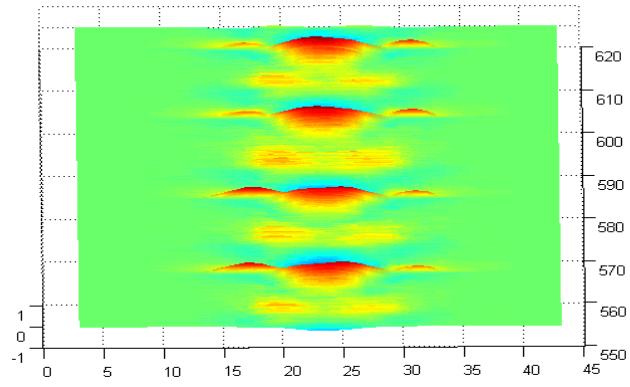




Limitation of high performance

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Kinetic effect of fast electron coupled with $m=1$ internal kink mode?



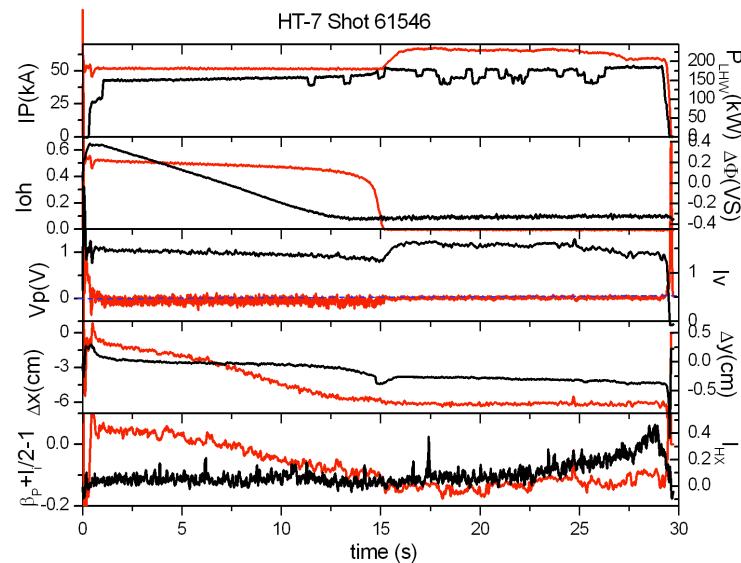
SVD analysis shows: ST, $m=1$ kink mode with small $m=2$ tearing



Discharge w/o ohmic current

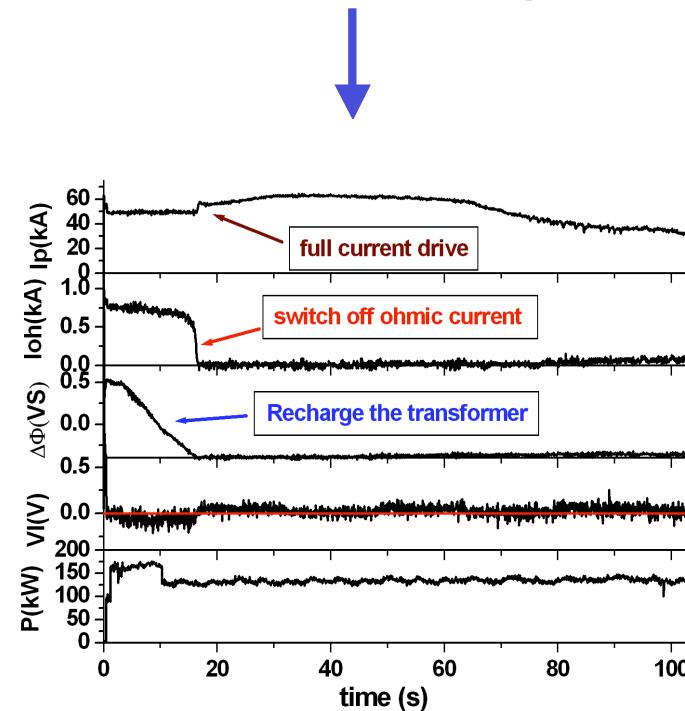
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Over LHCD recharged the transformer, the current in central solenoid was switched off when the transformer was reverse saturated



In 2003, sustained for 28 s
Density build-up terminated discharge

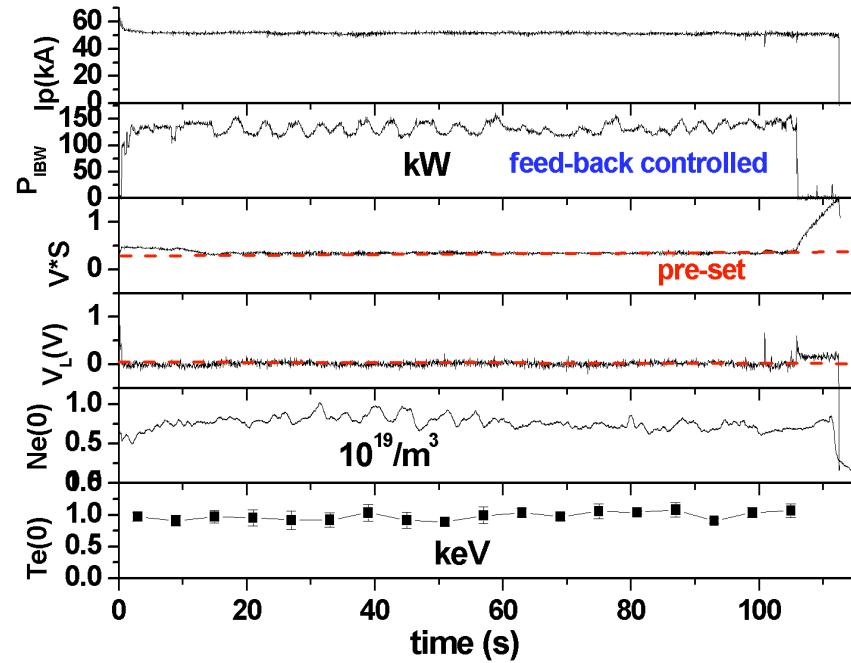
In 2004, sustained for 80s
Reflection protection terminated discharge



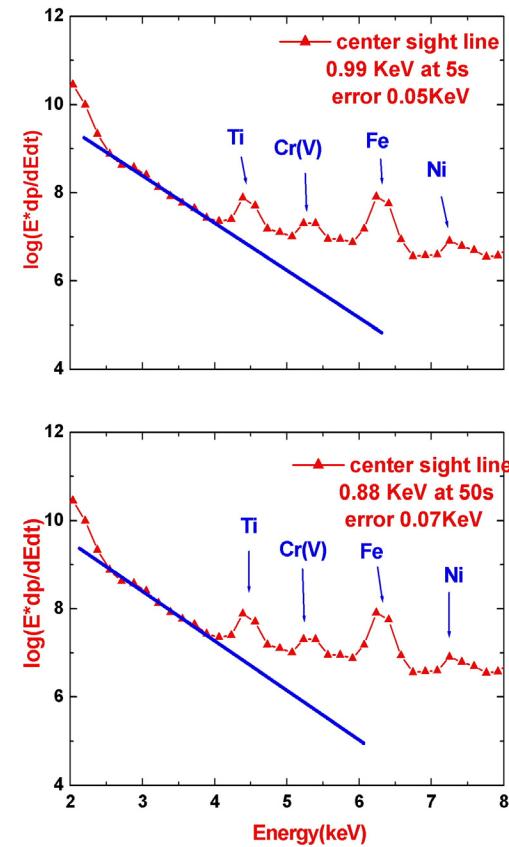


Long pulse discharges in 2004 (mode 2)

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Power feed back control by regulating
high voltage power

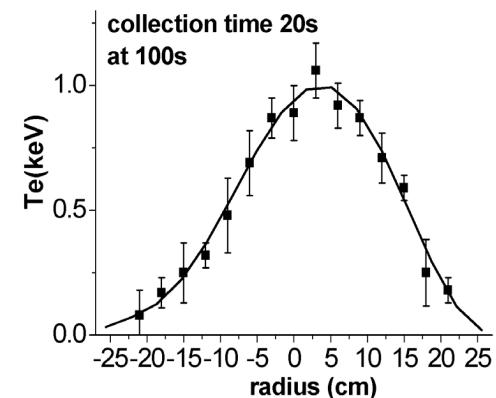
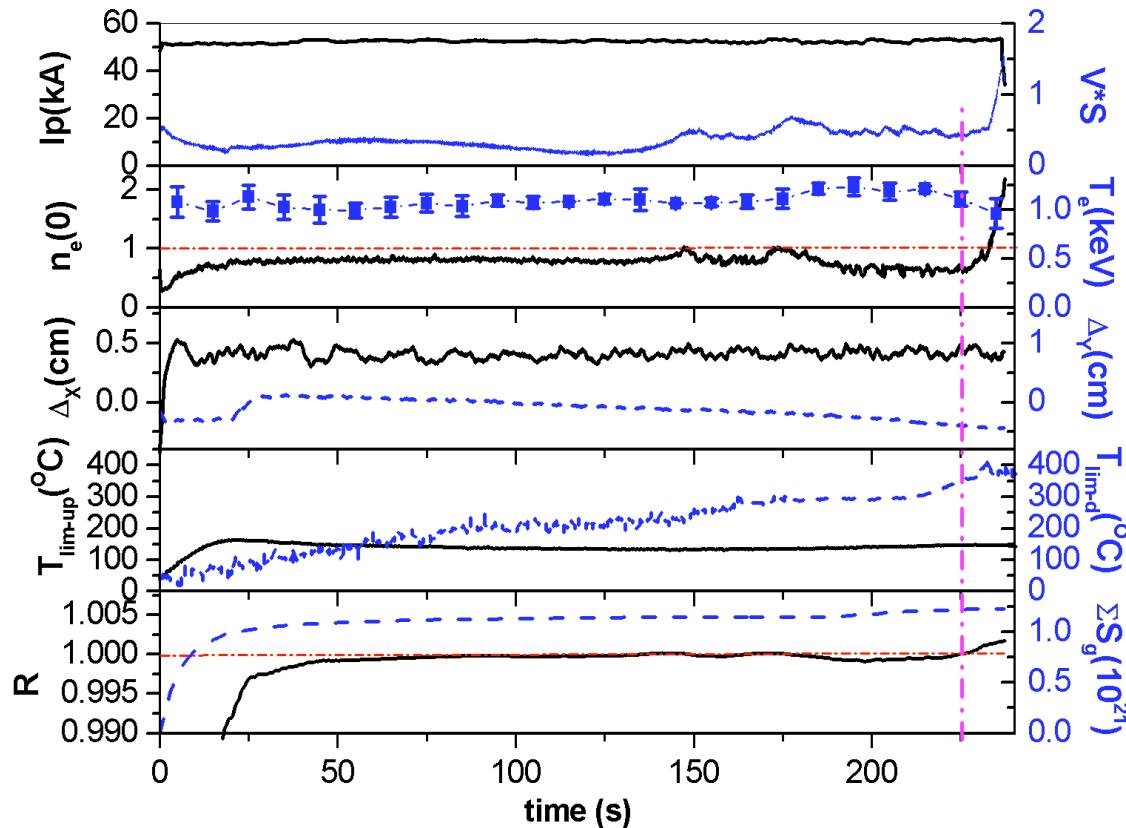


Metal impurity problem
alleviated



Long pulse discharges 4mins in 2004

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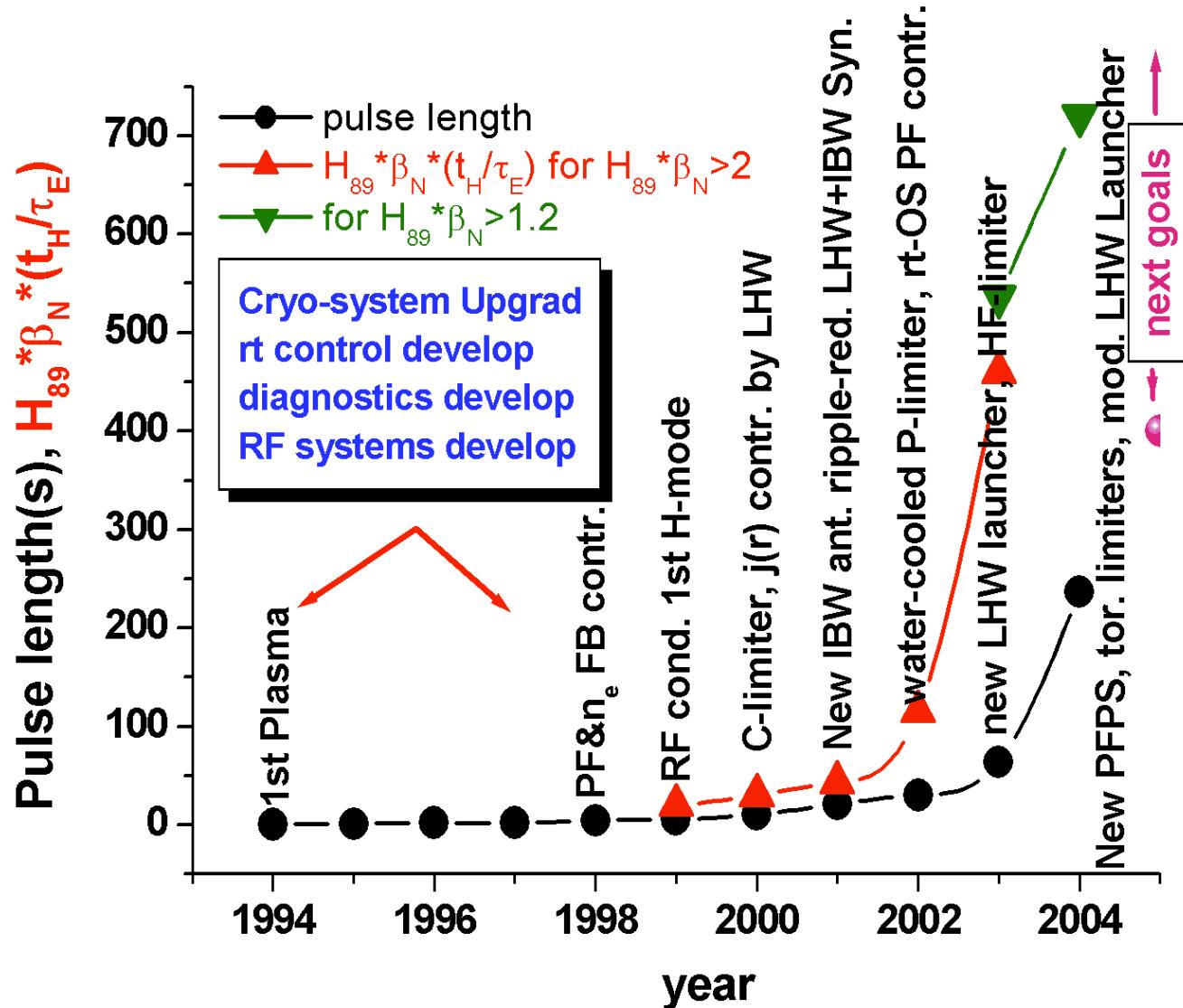
- $I_p \geq 50$ kA, $n_e(0) \sim 0.8 \times 10^{19}/\text{m}^3$, $T_{e0} \sim 1$ keV
- Wall saturation at ~ 80 s and refreshed at ~ 180 s
- Limiter temperature rise caused uncontrollable density at 225 s

Power feed back control by switching on/off spare klystrons



Milestones in HT-7

HT-7





Future plan

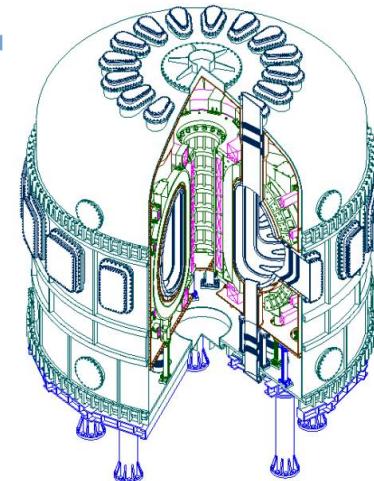
- TF coil cooling at 3.8k, for routinely $B_T = 2.5$ T .
- New operation regime at $I_P=250$ kA .
- Total injection power $P > 2$ MW .

To investigate:

- Plasma and wave interaction.
- Materials for PFC.
- Plasma confinement and transport.
- Simulation of advanced scenario.
- Advanced plasma diagnostics.
- Advanced plasma control.

Oriented to support the EAST project both scientifically and technically.

HT-7&EAST



Major Radius R_o 1.75-1.9 m
Minor Radius a 0.4-0.5 m
Toroidal Field B_o 3.5-4 T
Plasma Current I_P 1-1.5 MA

Elongation K_x 1.6 - 2
Triangularity d_x 0.4-0.8

Pulse length 1000 s

Heating and Driving:
(first phase)

ICRF	3 MW CW
LHCD	3.5 MW CW
ECRH	0.5 MW

Configuration:

Single null divertor
Double-null divertor
near double null



Summary

HT-7

- HT-7 experiments are strongly oriented to steady-state high performance plasmas and long pulse discharges.
- Technical improvements made great progress in achieving high performance plasmas under steady-state condition.
- Various scenarios of high performance plasma discharges (including edge H-mode, RS mode, high li mode etc) were realized.
- Stationary high performance plasma with $H_{89} * \beta_N > 2$ has been sustained for $> 220 \tau_E$ and $> 20 \tau_{CR}$.
- Long pulse discharge up to 4mins is successful with the new up-down toroidal belt limiters.
- Further experiments in HT-7 are strongly oriented to support the EAST project both scientifically and technically.